Therefore, renewal of the proposed 1.0 volt interim plugging criteria does not adversely affect steam generator tube integrity and results in acceptable dose consequences. The proposed amendment does not result in any increase in the probability or consequences of an accident previously evaluated within the Catawba Unit FSAR [Final Safety Analysis Report].

(2) The proposed license amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Renewal of the proposed steam generator tube interim plugging criteria does not introduce any significant changes to the plant design basis. Use of the criteria does not provide a mechanism which could result in an accident outside of the region of the tube support plat elevations—no ODSCC is occurring outside the thickness of the tube support plates. Neither a single or multiple tube rupture event would be expected in a steam generator in which the plugging criteria has been applied (during all plant conditions).

Upon application of the interim plugging criteria, no primary to secondary leakage during normal operation is anticipated during all plant conditions due to degradation at the tube support plate elevations in the Catawba Unit 1 steam generators. However, additional conservatism is built into the existing operating leakage limit with regard to protection against the maximum permissible single crack length which may be achieved during operation due, in large part, to the potential occurrence of through-wall cracks at locations other than the tube support plate intersections.

Application of the 1.0 volt interim steam generator tube plugging criteria at Catawba Unit 1 is not expected to result in tube burst during all plant conditions during operation. Tube burst margins are expected to meet RG 1.121 acceptance criteria. The limiting consequence of the application of the interim plugging criteria is a potential for SLB leakage. The methodology for calculating SLB leak rate uses a voltage-to-leakage correlation and this methodology has previously been reviewed and approved by the NRC. The SLB leakage value will be confirmed to be less than allowable levels prior to return to power of Catawba Unit 1. No unacceptable leakage is anticipated at normal operating or RCP locked rotor conditions.

Therefore, as the existing tube integrity criteria and accident analyses assumptions and results will continue to be met, the proposed license amendment does not create the possibility at a new or different kind of accident from any previously evaluated.

(3) The proposed license amendment does not involve a significant reduction in [a] margin of safety.

The use of the voltage based bobbin probe interim tube support plate elevation plugging criteria at Catawba Unit 1 is demonstrated to maintain steam generator tube integrity commensurate with the criteria of Regulatory Guide 1.121. [Regulatory Guide] 1.121 describes a method acceptable to the NRC staff for meeting GDCs [General Design Criteria] 14, 15, 31, and 32, by reducing the probability or the consequences of steam generator tube rupture. This is accomplished by determining the limiting conditions of degradation of steam generator tubing, as established by inservice inspection, for which tubes with unacceptable cracking should be removed from service. Implementation of the bobbin probe voltage based interim tube plugging criteria of 1.0 volt is supplemented by enhanced eddy current inspection guidelines to provide consistency in voltage normalization, a 100% eddy current inspection at the tube support plate elevations, and rotating pancake coil inspection requirements for the larger indications left in service to characterize the principle degradation as ODSCC. Even under the worst case conditions, the occurrence of ODSCC at the tube support plate elevations is not expected to lead to a steam generator tube rupture event during normal or faulted plant conditions.

Based on the analyses for Cycle 8, the expected leakage values and the leakage conditions required to be confirmed during accidents creating high differential pressures across the steam generator tubes (e.g. SLB), dose analysis confirm the maximum permissible leakage will result in offsite dose consequences within the guideline values. [An] MSLB accident with assumed leakage growth in the faulted generator results in the EAB and LPZ doses remaining within 10% of the 10 CFR 100 values of 25 Rem whole body and 300 Rem thyroid for the accident-initiated iodine spike, and 10 CFR 100 values for the pre-accident iodine spike.

The distribution of crack indications at the tube support plate elevations will be confirmed to result in acceptable primary to secondary leakage during all plant conditions and that radiological consequences are not adversely impacted.

Renewal of the tube support plate elevation plugging criteria for operation at Catawba Unit 1 will decrease the number of tubes which must be repaired by sleeving or taken out of service by plugging. The installation of steam generator tube plugs reduce the RCS flow margin. Thus, implementation of the alternate plugging criteria will maintain the margin of flow that would otherwise be reduced in the event of increased tube plugging.

Based on the above, it is concluded that the proposed license amendment requested does not result in a significant reduction in margin with respect to plant safety as defined in the Final Safety Analysis Report or any Bases of the plant Technical Specifications.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act on a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public and State comments received. Should the Commission takes this action, it will publish in the **Federal Register** a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of the **Federal Register** notice. Written comments may also be delivered to Room T–6 D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By March 13, 1995, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's ''Rules of Practice for Domestic Licensing Proceedings'' in 10 CFR part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Buidling, 2120 L Street, NW., Washington, DC, and at the local public document room located at the York County Library, 138 East Black Street, Rock Hill, South Carolina. If a request for a hearing or petition for leave to intervene is filed by the above date, the