Alternative Use of Resources

This action does not involve the use of any resources not previously considered in the Final Environmental Statement for the Millstone Nuclear Power Station, Unit No. 3.

Agencies and Persons Consulted

In accordance with its stated policy, the staff consulted with the Connecticut State official regarding the environmental impact of the proposed action. The State official had no comments.

Finding of No Significant Impact

Based upon the environmental assessment, the Commission concludes that the proposed action will not have a significant effect on the quality of the human environment. Accordingly, the Commission has determined not to prepare an environmental impact statement for the proposed action.

For further details with respect to the proposed action, see the licensee's letter dated August 16, 1994, as supplemented by letter dated January 10, 1995, which are available for public inspection at the Commission's Public Document Room, The Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room located at the Learning Resource Center, Three Rivers Community-Technical College, Thames Valley Campus, 574 New London Turnpike, Norwich, CT 06360.

Dated at Rockville, Maryland, this 3rd day of February 1995.

For the Nuclear Regulatory Commission **Phillip F. McKee**,

Director, Project Directorate I-4, Division of Reactor Projects—I/II, Office of Nuclear Reactor Regulation.

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[Docket No. 50-413]

Duke Power Company, et al.; Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of an amendment to Facility Operating License No. NPF– 35 issued to Duke Power Company (the licensee) for operation of the Catawba Nuclear Station, Unit 1, located in York County, South Carolina.

The proposed amendment request would propose the renewal for Catawba Unit 1 Cycle 9 operation of the steam generator tube inspection bobbin probe

voltage-based interim plugging criteria that had been previously approved for Cycle 8. Approval of this amendment will preclude unnecessary plugging or repairing tubes by sleeving due to the occurrence of outer diameter initiated stress corrosion cracking (ODSCC) at the tube support plate elevations in the Catawba Unit 1 steam generators. The interim plugging criteria approved for Cycle 8 and contained in the draft Generic Letter 94-XX, "Voltage-Based Repair Criteria for the Repair of Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking," can be summarized as follows:

Flaw indications with a bobbin coil voltage less than or equal to 1.0 volt can remain in service without further action. For flaw indications in excess of 1.0 volt but less than 2.7 volts, the tube can remain in service provided an RPC inspection of the indication does not detect ODSCC or any other degradation mode. Crack indications above 2.7 volts will be plugged or repaired by sleeving, and do not require RPC confirmation.

This amendment request reflects the "Requested Actions: for a licensee that chooses to implement a steam generator tube interim plugging criteria, as stated in the draft NRC Generic Letter, 94–XX "Voltage-Based Repair Criteria for the Repair of Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking."

The changes being proposed to the Technical Specification (TS) do not alter the interim plugging criteria currently stated in the TS which was approved and utilized during Cycle 8. The primary change to the TS is to incorporate the guidance of draft Generic Letter 94–XX, "Voltage-Based Repair Criteria for the Repair of Westinghouse Steam Generator Tubes Affected by Outside Diameter Stress Corrosion Cracking," which will allow removal of the cycle-specific limitation currently in the TS.

Before issuance of the proposed license amendment, the Commission will have made findings required by the Atomic Energy Act of 1954, as amended (the Act) and the Commission's regulations.

The Commission has made a proposed determination that the amendment request involves no significant hazards consideration. Under the Commission's regulations in 10 CFR 50.92, this means that operation of the facility in accordance with the proposed amendment would not (1) involve a significant increase in the probability or consequences of an accident previously evaluated; or (2) create the possibility of a new or different kind of accident from

any accident previously evaluated; or (3) involve a significant reduction in a margin of safety. As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) Operation of Catawba Unit 1 in accordance with the proposed license amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

A single tube rupture is not anticipated during operation of Catawba Unit 1. Based on the existing data base, the limiting RG [Regulatory Guide] 1.121 criterion for tube burst capability of 3 times normal operating differential is satisfied with 3/4" diameter tubing with bobbin coil indications with signal amplitudes less than 4.54 volts, regardless of the indicated depth measurement. This structural limit is based on a lower 95% prediction bound of the data and using LTL material properties. A 1.0 volt plugging criteria compares favorably with the structural limit considering the previously calculated growth rates for ODSCC within the Catawba Unit 1 steam generators. Assuming a voltage increase of 0.4 volts, and adding a 14% NDE uncertainty of 0.14 volts (90% cumulative probability) to the interim plugging criteria [IPC] of 1.0 volt results in an EOC [end-of-cycle] voltage of approximately 1.6 volts. This end of cycle voltage compares favorably with the Structural Limit of 4.54 volts. The applicability of assumed growth rates for each cycle of operation will be confirmed prior to return to power of Catawba Unit 1. A similar structural margin is anticipated for subsequent cycles.

In addition, for an EOC voltage structural limit of 4.54 volts, applying the 40% growth allowance and the 14% NDE uncertainty results in a margin between the structural limit and the alternate repair limit (2.7 volts), which is well within the structural limit. This repair limit will be applied for IPC implementation to repair bobbin indications greater than 2.7 volts independent of RPC confirmation of the indication.

Concerning SLB [steamline break] leakage in support of implementation to the interim plugging criteria, it will be determined whether the distribution of cracking indications at the tube support plate intersections at the end of a cycle are projected to be such that primary to secondary leakage would result in site boundary doses within the pertinent 10 CFR 100 limits. The SLB leakage rate calculation methodology * * * will be used to calculate End of Cycle SLB leakage. Based on EOC 8 projections, it is calculated that leakage during a postulated SLB event at the EOC 8 will be limited to approximately 1.61 gpm which is shown to result in acceptable dose consequences. [An] SLB leakage of 17.5 gpm in the faulted loop results in dose consequences which are less than the pertinent 10 CFR 100 limits. Similar results are expected for subsequent cycles and confirmation of leak rates will be performed prior to placing the [s]team generators in service.