

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to Technical Specification 3.9.4 would allow the containment personnel airlock to be open during fuel movement and core alterations. The containment personnel airlock is currently closed during fuel movement and core alterations to prevent the escape of radioactive material in the event of a fuel handling accident. The containment personnel airlock is not an initiator of any accident. Whether the containment personnel airlock doors are open or closed during fuel movement and core alterations has no effect on the probability of any accident previously evaluated.

The proposed change does alter assumptions previously made in evaluating the radiological consequences of the fuel handling accident inside the containment building. The proposed change allows for the containment personnel airlock to be open during refueling. The radiological consequences described in this change are bounded by those given in the Wolf Creek Generating Station Safety Evaluation Report and General Design Criteria 19. All doses for the proposed change are less than the acceptance criteria, therefore, there is no significant increase in the consequences of an accident previously analyzed.

The proposed change would significantly reduce the dose to workers in the containment in the event of a fuel handling accident by accelerating the containment evacuation process. The proposed change would also significantly decrease the wear on the containment personnel airlock doors and, consequently, increase the reliability of the containment personnel airlock doors in the event of an accident.

Since the probability of a fuel handling accident is unaffected by the airlock door positions, and the increased doses do not exceed acceptance limits, operation of the facility in accordance with the proposed amendment would not affect the probability or consequences of an accident previously analyzed.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change affects a previously evaluated accident, e.g., a fuel handling accident inside containment. The existing accident has been modified to account for the containment personnel airlock doors being opened at the time of the accident. It does not represent a significant change in the configuration or operation of the plant. Therefore, operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The margin of safety is reduced when the offsite and control room doses exceed the acceptance criteria in the Wolf Creek Generating Station Safety Evaluation Report. As previously discussed in the response to Standard I, the offsite and control room doses are below the acceptance criteria. Therefore, operation of the facility in accordance with the proposed amendment would not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room locations:* Emporia State University, William Allen White Library, 1200 Commercial Street, Emporia, Kansas 66801 and Washburn University School of Law Library, Topeka, Kansas 66621  
*Attorney for licensee:* Jay Silberg, Esq., Shaw, Pittman, Potts and Trowbridge, 2300 N Street, N.W., Washington, D.C. 20037

*NRC Project Director:* William H. Bateman

Previously Published Notices Of Consideration Of Issuance Of Amendments To Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, And Opportunity For A Hearing

The following notices were previously published as separate individual notices. The notice content was the same as above. They were published as individual notices either because time did not allow the Commission to wait for this biweekly notice or because the action involved exigent circumstances. They are repeated here because the biweekly notice lists all amendments issued or proposed to be issued involving no significant hazards consideration.

For details, see the individual notice in the Federal Register on the day and page cited. This notice does not extend the notice period of the original notice.

Commonwealth Edison Company, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois; Docket Nos. 50-254 and 50-265, Quad Cities Nuclear Power Station, Units 1 and 2, Rock Island County, Illinois

*Date of amendment request:* November 14, 1995

*Description of amendment request:* The proposed amendment would close out additional open items identified in

the NRC staff's review of the upgrade of the Dresden and Quad Cities Technical Specifications (TS) to the standard Technical Specifications (STS) contained in NUREG-0123. The Technical Specification Upgrade Program (TSUP) is not a complete adaption of the STS. The TS upgrade focuses on (1) integrating additional information such as equipment operability requirements during shutdown conditions, (2) clarifying requirements such as limiting conditions for operation and action statements utilizing STS terminology, (3) deleting superseded requirements and modifications to the TS based on the licensee's responses to Generic Letter (GL), and (4) relocating specific items to more appropriate TS locations. The November 14, 1995, application proposed to close out all open items identified during the NRC's review as noted in previous NRC staff Safety Evaluations for previously provided submittals regarding the TSUP project.

*Date of publication of individual notice in Federal Register:* November 29, 1995 (60 FR 61272).

*Expiration date of individual notice:* December 28, 1995

*Local Public Document Room location:* for Dresden, the Morris Area Public Library District, 604 Liberty Street, Morris, Illinois; and for Quad Cities Station, the Dixon Public Library, 221 Hennepin Avenue, Dixon, Illinois.

Commonwealth Edison Company, Docket Nos. 50-237 and 50-249, Dresden Nuclear Power Station, Units 2 and 3, Grundy County, Illinois; Docket Nos. 50-373 and 50-374, LaSalle County Station, Units 1 and 2, LaSalle County, Illinois; Docket Nos. 50-254 and 50-265, Quad Cities Nuclear Power Station, Units 1 and 2, Rock Island County, Illinois

*Date of amendment request:* November 14, 1995

*Description of amendment request:* The proposed amendment would change the technical specifications of these plants to incorporate 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing For Water-Cooled Power Reactors", Option B.

*Date of publication of individual notice in Federal Register:* December 7, 1995 (60 FR 62896)

*Expiration date of individual notice:* January 8, 1996

*Local Public Document Room location:* for Dresden Station, Morris Area Public Library District, 604 Liberty Street, Morris, Illinois; for LaSalle County Station, Jacobs Memorial Library, Illinois Valley Community