

LOCA does not change from those currently resulting from a LOCA initiated while in TS 3.5.2 ACTION statement (a.), thus, there is no change in consequences of an accident previously evaluated in the FSAR.

2. The possibility of an accident or a malfunction of a different type than any previously evaluated is not created.

The proposed TSCR [TS change request] only results in a one time increase in the allowable outage time for each train of RHR. It does not result in an operational condition different from that which has already been considered by TS. Therefore, the change does not create the possibility of a new or different kind of accident or malfunction.

3. The margin of safety has not been significantly reduced.

The effects of increasing the allowed outage time on the calculated core damage frequency has been evaluated and determined to be small.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Fairfield County Library, 300 Washington Street, Winnsboro, SC 29180

Attorney for licensee: Randolph R. Mahan, South Carolina Electric & Gas Company, Post Office Box 764, Columbia, South Carolina 29218

NRC Project Director: Frederick J. Hebdon

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50-440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of amendment request:
November 22, 1995

Description of amendment request:
The proposed amendment would change the operating license to reflect the license transfer for part of Ohio Edison Company's ownership interest in the Perry Nuclear Power Plant (PNPP), Unit No. 1 to its wholly owned subsidiary, OES Nuclear Inc.

Basis for proposed no significant hazards consideration determination:
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes to the PNPP Operating License are administrative and

have no effect on the PNPP facility, programs, personnel or any plant systems. All Limiting Conditions for Operation, Limiting Safety Systems Settings, and Safety Limits specified in the Technical Specifications will remain unchanged. This change meets one of the examples of a change not likely to involve a significant hazards consideration in that it is a purely administrative change. 48 Fed. Reg. 14,864 (1983).

2. The proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to the PNPP Operating License are administrative and have no effect on the PNPP facility, programs, personnel or any plant systems. PNPP's design and design bases will remain unchanged as will All Limiting Conditions for Operation, Limiting Safety Systems Settings, and Safety Limits specified in the Technical Specifications. This change meets one of the examples of a change not likely to involve a significant hazards consideration in that it is a purely administrative change. 48 Fed. Reg. 14,864 (1983).

3. The proposed changes do not involve a significant reduction in the margin of safety.

The proposed changes to the PNPP Operating License are administrative and have no effect on the PNPP facility, programs, personnel or any plant systems. All Limiting Conditions for Operation, Limiting Safety Systems Settings, and Safety Limits specified in the Technical Specifications will remain unchanged. This change meets one of the examples of a change not likely to involve a significant hazards consideration in that it is a purely administrative change. 48 Fed. Reg. 14,864 (1983).

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Perry Public Library, 3753 Main Street, Perry, Ohio 44081

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037

NRC Project Director: Gail H. Marcus
Virginia Electric and Power Company, Docket Nos. 50-338 and 50-339, North Anna Power Station, Units No. 1 and No. 2, Louisa County, Virginia

Date of amendment request:
November 20, 1995

Description of amendment request:
The proposed changes would revise the Technical Specifications (TS) for the North Anna Power Station, Units No. 1 and No. 2 (NA-1&2). Specifically, the change would permit the use of 10 CFR Part 50, Appendix J, Option B,

Performance-Based Containment Leakage Rate Testing.

The Nuclear Regulatory Commission (NRC) has amended its regulations to provide a performance-based option for leakage-rate testing of containments. This testing option is available in lieu of compliance with the prescriptive requirements contained in Appendix J regulations. In order to implement the performance-based leakage-rate testing option the TS must be changed to eliminate reference to the prescriptive Appendix J requirements. Therefore, the licensee is proposing a change to the NA-1&2 TS to eliminate the current prescriptive requirements for leakage rate testing of the containment and reference Option B to 10 CFR 50 Appendix J and NRC Regulatory Guide 1.163, "Performance-Based Containment Leakage-Test Program." This change will permit use of the performance-based surveillance testing, Option B, of 10 CFR 50 Appendix J.

Basis for proposed no significant hazards consideration determination:
As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Specifically, operation of North Anna Power Station with the proposed change will not:

1. Involve a significant increase in either the probability of occurrence or consequences of any accident or equipment malfunction scenario which is important to safety and which has been previously evaluated in the Updated Final Safety Analysis Report (UFSAR).

Plant systems and components will not be operated in a different manner as a result of the proposed Technical Specifications change. The proposed change permits a performance-based approach to determining the leakage-rate test frequency for the containment and containment penetrations (Type A, B, and C tests). Since the proposed change only affects the test frequency for containment and containment penetrations, the probability of occurrence of an accident is not affected by the proposed changes in the leak-rate test interval.

The proposed change increases the probability of a malfunction due to the longer intervals between leakage tests. It has been estimated that the longer test intervals will increase the overall accident risk to the public by approximately 0.7% and 2.2% (for changes in the frequency of Type A tests and Type B and C tests, respectively). However, this increase in accident risk has been judged to be insignificant. This increase has been reviewed and judged to be acceptable by the NRC as documented in NUREG-1493 and the recent rulemaking to 10 CFR 50 Appendix J.

The Limiting Conditions for Operation are not being changed for the containment or any other safety system. The containment and other safety system remain operable as