

*NRC Project Director:* Phillip F. McKee

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

*Date of amendment request:* August 17, 1995

*Description of amendment request:* The Commission issued Amendment Nos. 128 and 122 to the Facility Operating Licenses for Catawba Units 1 and 2 on February 17, 1995, which revised Technical Specification (TS) Table 2.2-1 and TS Surveillance Requirement (SR) 4.2.5 to allow a change in the method for measuring reactor coolant system (RCS) flowrate from the calorimetric heat balance method to a method based on a one-time calibration of the RCS cold leg elbow differential pressure taps. In its application submitted on January 10, 1994, for the above listed amendments, Duke Power (the licensee) neglected to modify SR 4.2.5.2 to delete that portion of the SR that specifies that the measurement instrumentation shall be calibrated within 7 days prior to the performance of the flowrate measurement. The licensee states that the requirement to calibrate the measurement instrumentation within 7 days prior to the performance of the flowrate measurement is impractical based on utilization of the cold leg elbow pressure tap method of RCS flowrate measurement. Accordingly, the licensee proposes to modify SR 4.2.5.2 to reflect the deletion of the subject requirement.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

*Criterion 1*

The requested amendments will not involve a significant increase in the probability or consequences of an accident previously evaluated. This change is considered administrative in nature and should have been requested in Duke Power Company's January 10, 1994 application, as amended. The instrumentation which was subject to the requirement is no longer utilized in the fulfillment of the TS required RCS flowrate determination. The proposed changes will not result in any impact upon accident probabilities, since the RCS flowrate measurement instrumentation is not accident initiating equipment. Likewise, they will not result in any impact upon accident consequences, since no change to any method or frequency of calibration of the RCS flowrate transmitters will result. The plant response to accidents will not be affected.

*Criterion 2*

The requested amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated. No change is being made to any plant design feature, or to the manner in which the plant will be operated. Therefore, no new accident causal mechanisms can be generated. As noted above, the proposed changes are considered administrative in nature, and should have been requested in the January 10, 1994 application, as amended.

*Criterion 3*

The requested amendments will not involve a significant reduction in a margin of safety. No impact upon any fission product barriers will occur as a result of the approval of the proposed changes. No change to plant design, operating, maintenance, or test characteristics will result from the proposed amendments. No impact upon any plant safety margins will result.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room location:* York County Library, 138 East Black Street, Rock Hill, South Carolina 29730

*Attorney for licensee:* Mr. Albert Carr, Duke Power Company, 422 South Church Street, Charlotte, North Carolina 28242

*NRC Project Director:* Herbert N. Berkow

Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina

*Date of amendment request:* November 15, 1995

*Description of amendment request:* The proposed amendments modify Technical Specification (TS) 3/4.7.1 and the associated Bases to increase the setpoint tolerance of the main steam safety valves (MSSVs) from plus or minus one percent to plus or minus three percent, to incorporate a requirement to reset as-left MSSV lift settings to within plus or minus one percent following surveillance testing, and to delete two obsolete footnotes.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

*Criterion 1*

The requested amendments will not involve a significant increase in the probability or consequences of an accident

previously evaluated. As demonstrated previously, all applicable licensing basis safety analyses were evaluated with a MSSV setpoint drift of plus or minus 3%. The results of the evaluations were within all appropriate accident analysis acceptance criteria. No significant impact on DNBR results, peak primary or secondary pressures, peak fuel cladding temperature, dose, or any other accident analysis acceptance criterion was involved. No impact on the probability of any accident occurring exists as a result of the increased MSSV setpoint tolerance.

*Criterion 2*

The requested amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated. No change is being made to any plant design feature, or to the manner in which the plant will be operated. Therefore, no new accident causal mechanisms can be generated. The MSSV setpoint tolerance only affects the time at which the valve opens following or during a transient, and is not a contributor to the probability of an accident.

*Criterion 3*

The requested amendments will not involve a significant reduction in a margin of safety. As stated above, all relevant accident analyses were examined to determine the effect of the wider MSSV setpoint tolerances. All analysis results are within applicable acceptance criteria. Finally, the NRC has previously approved TS changes for other plants seeking to use the [plus or minus] 3 [percent] setpoint tolerance, including McGuire Nuclear Station (reference Amendment Nos. 146 and 128 for Units 1 and 2, respectively).

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*Date of amendment request:* November 15, 1995

*Description of amendment request:* The proposed amendments modify Technical Specification (TS) Limiting Condition for Operation 3.7.5 to raise the minimum nuclear service water system's (RN) water level in the standby nuclear service water pond (SNSWP) from 570 to 571 feet mean sea level.