The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Callaway County Public Library, 710 Court Street, Fulton, Missouri 65251.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts & Trowbridge, 2300 N Street, N.W., Washington, DC 20037.

NRC Project Director: Leif J.

Norrholm.

Union Electric Company, Docket No. 50–483, Callaway Plant, Unit 1, Callaway County, Missouri

Date of amendment request: December 9, 1994, as supplemented on December 22, 1994.

Description of amendment request: The proposed amendment would revise Technical Specification (TS) Surveillance Requirement 4.8.1.1.2f.7 to remove the requirement to perform the hot restart test within 5 minutes of completing the 24-hour endurance test and place that requirement in a separate TS.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The proposed revision does not involve a significant hazards consideration because operation of Callaway Plant with this change would not:

(1) Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed revision to the T/S will not adversely impact plant safety since the requirement to perform the hot restart test will still be implemented via a separate surveillance requirement that demonstrates the hot restart functional capability of the diesel generators.

(2) Create the possibility of a new or different kind of accident from any previously evaluated.

There are no design changes being made that would create a new type of accident or malfunction and the method and manner of plant operation remain unchanged. The performance capability of the emergency diesel generators will not be affected. The verification of the hot restart capability of the diesel generators will still be performed, only the timing of the performance will be changed to give plant operators added flexibility and prevent critical path complications during outages.

(3) Involve a significant reduction in a margin of safety.

There are no changes being made to the safety limits or safety system settings that would adversely impact plant safety. The diesel generators will still perform their intended safety function following a loss of offsite power, to achieve and maintain the plant in a safe shutdown condition.

Based on the above discussions, it has been determined that the requested Technical Specification change does not involve a significant increase in the probability or consequences of an accident or create the possibility of a new or different kind of accident or condition over previous evaluations; or involve a significant reduction in a margin of safety. Therefore, the requested license amendment does not involve a significant hazards consideration.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Callaway County Public Library, 710 Court Street, Fulton, Missouri 65251.

Attorney for licensee: Gerald Charnoff, Esq., Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037. NRC Project Director: Leif J. Norrholm.

Vermont Yankee Nuclear Power Corporation, Docket No. 50–271, Vermont Yankee Nuclear Power Station, Vernon, Vermont

Date of amendment request: December 14, 1994.

Description of amendment request: The proposed amendment would revise instrument identification for low reactor pressure instrument trip cards in emergency core cooling system (ECCS) actuation to reflect a design change to be installed during the 1995 refueling outage.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change to the identification numbers for certain reactor pressure instrumentation as included in the Technical Specifications for ECCS Actuation Instrumentation is only necessary because the specific identification numbers (Tag Nos.) have been listed in the [***]. This is considered an administrative type change. Acceptable measurement of Low Reactor Pressure is still assured. All automatic control or trip functions will continue to be provided.

The proposed change does not result in any function or setpoint change. The

hardware changes which have resulted in a need to change the Technical Specifications have removed instrumentation no longer required to be installed in the circuitry for measuring ECCS Low Reactor Pressure. The existing logic for Low Reactor Pressure will remain the same. The only change applicable to implementation of the design modification is the use of different trip cards to provide the trip function for ECCS Low Reactor Pressure.

The requested change to ECCS Actuation Instrumentation Tables does not impact any FSAR [Final Safety Analysis Report] safety analysis involving the ECCS or Protection Systems. These measurement functions are not contributors to the initiation of accidents.

The change in instrument Tag Nos. on Tables 3.2.1 and 4.2.1 will have no affect on any safety limit setting or plant system operation and, therefore, does not modify or add any initiating parameters that would significantly increase the probability or consequences of any previously analyzed accident.

The administrative change to correct a typographical error on Table 4.2.1 will have no affect on plant hardware, plant design, safety limit setting or plant system operation and, therefore, does not modify or add any initiating parameters that would significantly increase the probability or consequences of any previously analyzed accident.

Therefore, it is concluded that there is not a significant increase in the probability or consequence of an accident previously evaluated.

2. The proposal to change instrument Tag Nos. does not result in any function changes or changes to Technical Specification requirements pertaining to these functions.

The proposed change does not involve any change in Technical Specification trip setpoints, plant operation, redundancy, protective function or design basis of the plant. There is no impact on any existing safety analysis or safety design limits. Low Reactor Pressure instrumentation functions do not initiate nuclear system parameter variations which are considered potential initiating causes of threats to the fuel and the nuclear system process barrier or that would create any new or different kind of accident.

As discussed above, the proposed administrative change only corrects a typographical error concerning equipment identification numbers. This change does not affect any equipment and it does not involve any potential initiating events that would create any new or different kind of accident.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposal to change the identification numbers for certain reactor pressure instrumentation as included in the Technical Specifications for ECCS Actuation Instrumentation does not affect any existing safety margins. The change by itself is administrative. The hardware changes which have resulted in a need to change the Technical Specifications have been reviewed per 10 CFR 50.59(a)(2) and determined to not constitute an unreviewed safety question.

The change in Tag Nos. or the change in the instrumentation used to measure low