The proposed amendment has been reviewed for acceptability at the stations considering similarity of system or component design affecting containment integrity. No new modes of operation are introduced by the proposed changes. Surveillance requirements are changed to reflect corresponding changes associated with Option B to 10 CFR part 50, Appendix J and improvements in technique or frequency of leak rate testing performance. The proposed changes maintain at least the present level of operability of any such system that affects plant containment integrity. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any previously evaluated.

The associated systems that affect plant leak rate integrity related to the proposed amendment, are not assumed in any plant safety analysis to initiate any accident sequence. In addition, the proposed surveillance requirements for any such affected systems are consistent with the current requirements specified within the Technical Specifications and are consistent with the requirements of Option B to 10 CFR part 50, Appendix J. The proposed surveillance requirements maintain an equivalent level of reliability and availability of all affected systems and therefore, does not increase the consequences of any previously evaluated accident. As such, the probability of the affected systems, associated with leak rate test integrity, from performing their intended function, is unaffected by the proposed limiting conditions for operation and surveillance requirements.

(3) Involve a significant reduction in the margin of safety because:

The provisions specified in Option B to 10 CFR part 50 Appendix J allows changes to Type A, Type B and Type C test intervals based upon the performance of past leak rate tests. The effect of extending containment leakage rate testing intervals is a corresponding increase in the likelihood of containment leakage.

The degree to which intervals can be extended is a direct function on the potential effect on existing plant safety margins and the public health and safety that can occur due to an increased likelihood of containment leakage.

Changing Appendix J test intervals from those currently provided in the Technical Specification to those provided for in 10 CFR part 50, Appendix J, Option B, slightly increases the risk associated with Type A, Type B, and Type C specific accident sequences. Historical data suggests that increasing the Type C test interval can slightly increase the associated risk; however, this is compensated by the corresponding risk reduction benefits associated with reduction in component cycling, stress, and wear associated with increased test intervals. In addition, when considering the total integrated risk which includes all analyzed accident sequences, the risk associated with increasing test intervals is negligible.

ComEd proposes to revise the Technical Specifications to be consistent with those provisions specified in Option B of 10 CFR part 50, Appendix J. The proposed changes

are consistent with current plant safety analyses. In addition, these proposed changes do not involve revisions to the design of the station. As such, the proposed individual changes will maintain the same level of reliability of the equipment associated with containment integrity, assumed to operate in the plant safety analysis, or provide continued assurance that specified parameters affecting plant leak rate integrity, will remain within their acceptance limits. Therefore, the proposed changes provide continued assurance of the leakage integrity of the containment without adversely affecting the public health and safety and as such, will not significantly reduce existing plant safety margins.

The proposed amendment to the Technical Specifications implements present requirements, or the requirements in accordance with the guidelines set forth in Option B to 10 CFR part 50, Appendix J. The proposed changes have been evaluated and found to be acceptable for use at the stations based on system design, safety analysis requirements, and operational performance. Since the proposed changes are based on NRC accepted provisions that are applicable at the stations and maintain necessary levels of system or component reliability affecting plant containment integrity, the proposed changes do not involve a significant reduction in the margin of safety.

The performance-based approach to leakage rate testing concludes that the impact on public health and safety due to revised testing intervals is negligible. The proposed amendment for the stations will not reduce the availability of systems associated with containment integrity when required to mitigate accident conditions; therefore, the proposed changes do not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

The Commission is seeking public comments on this proposed determination. Any comments received within 30 days after the date of publication of this notice will be considered in making any final determination.

Normally, the Commission will not issue the amendment until the expiration of the 30-day notice period. However, should circumstances change during the notice period such that failure to act in a timely way would result, for example, in derating or shutdown of the facility, the Commission may issue the license amendment before the expiration of the 30-day notice period, provided that its final determination is that the amendment involves no significant hazards consideration. The final determination will consider all public

and State comments received. Should the Commission take this action, it will publish in the Federal Register a notice of issuance and provide for opportunity for a hearing after issuance. The Commission expects that the need to take this action will occur very infrequently.

Written comments may be submitted by mail to the Rules Review and Directives Branch, Division of Freedom of Information and Publications Services, Office of Administration, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and should cite the publication date and page number of this Federal Register notice. Written comments may also be delivered to Room 6D22, Two White Flint North, 11545 Rockville Pike, Rockville, Maryland, from 7:30 a.m. to 4:15 p.m. Federal workdays. Copies of written comments received may be examined at the NRC Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC.

The filing of requests for hearing and petitions for leave to intervene is discussed below.

By January 8, 1996, the licensee may file a request for a hearing with respect to issuance of the amendment to the subject facility operating license and any person whose interest may be affected by this proceeding and who wishes to participate as a party in the proceeding must file a written request for a hearing and a petition for leave to intervene. Requests for a hearing and a petition for leave to intervene shall be filed in accordance with the Commission's "Rules of Practice for Domestic Licensing Proceedings" in 10 CFR Part 2. Interested persons should consult a current copy of 10 CFR 2.714 which is available at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW. Washington, DC, and at the local public document rooms located at the Morris Area Public Library District, 604 Liberty Street, Morris, Illinois for Dresden Station, Jacobs Memorial Library, Illinois Valley Community College, Oglesby, Illinois for LaSalle County Station, and Dixon Public Library, 221 Hennepin Avenue, Dixon, Illinois for Quad Cities Station. If a request for a hearing or petition for leave to intervene is filed by the above date, the Commission or an Atomic Safety and Licensing Board, designated by the Commission or by the Chairman of the Atomic Safety and Licensing Board Panel, will rule on the request and/or petition; and the Secretary or the designated Atomic Safety and Licensing Board will issue a notice of hearing or an appropriate order.