be no change in the analyzed accident scenario since even in the event of a loss of offsite power event, the safety functions would be completed. Thus, the consequences of any previously evaluated accident have not increased.

2. The proposed change does not create the possibility of a new or different kind of accident from any previously evaluated:

The proposed change introduces no new mode of plant operation and it does not involve physical modification to the plant. Therefore, it does not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety:

This change does not involve a significant reduction in a margin of safety since the proposed change maintains a safety related, diesel-backed power supply to these buses whether the power is supplied from the inverters or from the alternate power supply. If a loss of offsite power event were to occur while the buses were supplied from the alternate power source, the safety functions being performed by components supplied from these buses would occur. Thus, there has been no reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Perry Public Library, 3753 Main Street, Perry, Ohio 44081

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037

NRC Project Director: Gail H. Marcus

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50–440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of amendment request: November 2, 1995

Description of amendment request: The proposed amendment to the Perry Nuclear Power Plant Technical Specifications revises those specifications associated with handling irradiated fuel in Primary Containment and the Fuel Handling Building, and selected specifications associated with CORE ALTERATIONS. Specifically, analysis identifies that only —recently— irradiated fuel contains sufficient fission products to require OPERABILITY of accident mitigation features to meet the accident analysis assumptions. Analyses also show that accident mitigation features such as building INTEGRITY and engineered safety feature (ESF) ventilation systems are not required for CORE ALTERATION events.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed requirements are imposed during specific activities which can be postulated to result in significant radioactive releases. The proposed APPLICABILITY requirements are consistent with either the original design basis analyses or with revised analyses performed to support this proposed amendment. Because the equipment controlled by the revised Specifications is not considered an initiator to any previously analyzed accident, inoperability of the equipment cannot increase the probability of any previously evaluated accident.

Consistent with the original design basis analysis, the reanalysis concludes that radiological consequences of the fuel handling accident are well within the 10 CFR 100.11 limits, as defined by acceptance criteria in Standard Review Plan Section 15.7.4. The reanalysis has previously been submitted to the Nuclear Regulatory Commission for review, and NRC confirmatory calculations reached consistent results (reference NRC Safety Evaluation for License Amendment No. 35). The results of the CORE ALTERATION events other than the fuel handling accident remain unchanged from the original design basis, which showed that these events do not result in fuel cladding integrity damage or radioactive releases. Therefore, the proposed changes do not significantly increase the consequences of any previously evaluated accident.

Based on the above, the proposed changes do not significantly increase the probability or consequences of any accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed requirements are imposed when specific activities represent situations where significant radioactive releases can be postulated. The proposed APPLICABILITY requirements are consistent with design basis analyses. The proposed changes do not introduce any new modes of plant operation and do not involve physical modifications to the plant. Therefore, the proposed changes do not create the possibility of a new or different kind of accidident from any previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed change imposes controls to ensure that during performance of activities which represent situations where radioactive releases are postulated, the radiological consequences are at or below the established licensing limit. Safety margins and analytical conservatisms have been evaluated and are well understood. Substantial conservatism is retained to ensure that the analysis adequately bounds all postulated event scenarios. The current margin of safety is retained.

Specifically, the margin of safety for the fuel handling accident is the difference between the 10 CFR 100 limits and the licensing limit defined by the Standard Review Plan (NUREG 0800), Section 15.7.4. The licensing limit is defined by the Standard Review Plan as being —well within— the 10 CFR 100 limits, with ''well within'' defined as 25% of the 10 CFR 100 limits for the fuel handling accident. Excess margin is the difference between the postulated doses and the corresponding licensing limit. In the NRCs initial licensing review of the Perry Nuclear Power Plant (NUREG–0887, Section 15.3.3), the NRC accepted the design and analyses based on the results of the analyses being well within the guideline values of 10 CFR 100.

The proposed APPLICABILITY requirements continue to ensure that the whole-body and thyroid doses at the exclusion area and low population zone boundaries as well as control room doses are at or below the corresponding licensing limit. The margin of safety is unchanged; therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The margin of safety for the CORE ALTERATION events other than the fuel handling accident discussed above also remains the same as in the original design basis analyses, since the proposed changes do not impact on the Technical Specification requirements for systems needed to prevent or mitigate such CORE ALTERATION events.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Project Director: Gail H. Marcus

Notice Of Issuance Of Amendments To Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and