- Non-LOCA [loss-of-coolant accident]
 Events
 - Large Break and Small Break LOCA

Steam Generator Tube Rupture With the [delta]75 SGs and revised operating conditions, the calculated results (i.e., DNBR [departure from nucleate boiling ratio], Primary and Secondary System Pressure, Peak Clad Temperature, Metal Water Reaction, Challenge to Long Term Cooling, Environmental Conditions Inside and Outside containment, etc.) for the accidents are similar to those currently reported in the VCSNS FSAR and remain within applicable Regulatory Acceptance Criteria. Select results (i.e., Containment Pressure during a Steam Line Break, Minimum DNBR for Rod Withdrawal from Subcritical, etc.) are slightly more limiting than those currently reported in the FSAR due to the use of the assumed operating conditions with the [delta]75 SGs and in some cases, use of an uprated core power of 2900 MWt. However, in all cases, the calculated results do not challenge the integrity of the primary/secondary/ containment pressure boundary and remain within the regulatory acceptance criteria

applied to VCSNS's current licensing basis.
Given that calculated radiological
consequences are not significantly higher
than current FSAR results and remain well
within 10 CFR 100 limits, it is concluded that
the consequences of an accident previously
evaluated in the FSAR are not significantly

increased.

2. The proposed license amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

Uprate power operation will not introduce any new accident initiator mechanisms. Structural integrity of the RCS is maintained during all plant conditions through compliance with the ASME code and 10 CFR 50 Appendix G requirements. Design requirements of auxiliary systems are met with the RSGs [replacement steam generators] and uprate power operation. No new failure modes or limiting single failures have been identified. Since the safety and design requirements continue to be met and the integrity of the reactor coolant system pressure boundary is not challenged, no new accident scenarios have been created. Therefore, the types of accidents defined in the FSAR continue to represent the credible spectrum of events to be analyzed which determine safe plant operation.

3. The proposed license amendment does not involve a significant reduction in a margin of safety.

Although uprate power operation will require changes to the VCSNS Technical Specifications, the proposed changes are supported by extensive LOCA, NON-LOCA and SGTR [steam generator tube rupture] analyses. These analyses show acceptable consequences with margin to the applicable regulatory limits. All equipment required to function during accident conditions has been shown to remain qualified and thus will perform their design function, and all components remain in compliance with the codes and standards in effect when VCSNS was originally licensed (with the exception of

the replacement steam generators which use the 1986 ASME Code Section III Edition).

Low Temperature Overpressure transients which could challenge RCS structural integrity are not impacted by the revision to the Pressure Temperature Limitations Curves. The curves are not directly impacted, the changes do not reduce any margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Fairfield County Library, 300 Washington Street, Winnsboro, SC 29180

Attorney for licensee: Randolph R. Mahan, South Carolina Electric & Gas Company, Post Office Box 764, Columbia, South Carolina 29218

NRC Project Director: Frederick J. Hebdon

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50–440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of amendment request: August 29, 1995

Description of amendment request: The proposed amendment would revise the Technical Specifications for allowable values and trip setpoints for selected plant process instrumentation. The new allowable values/setpoints are in accordance with the instrument setpoint methodology accepted by the NRC staff in a letter dated July 18, 1995.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed revised Trip Setpoints and Allowable Values are more conservative than those currently approved in the Technical Specifications. Therefore, any proposed system or component actuations will occur earlier, resulting in a more conservative plant response. Thus, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change to the Technical Specifications does not introduce any new components nor does it modify the design of any existing components. Other than making Trip Setpoints and Allowable Values of existing instrumentation more conservative, the change does not affect the design or function of any plant system, structure, or component, nor does it change the way plant systems are operated. Thus, the possibility of a new or different kind of accident previously evaluated is not created.

3. The proposed change does not result in a significant reduction in the margin of safety.

Since the proposed revised Trip Setpoints and Allowable Values are more conservative than the existing values, the margin of safety would be increased by issuance of the changes. Thus, the proposed change does not result in a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Perry Public Library, 3753 Main Street, Perry, Ohio 44081

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037

NRC Project Director: Gail H. Marcus

The Cleveland Electric Illuminating Company, Centerior Service Company, Duquesne Light Company, Ohio Edison Company, Pennsylvania Power Company, Toledo Edison Company, Docket No. 50–440, Perry Nuclear Power Plant, Unit No. 1, Lake County, Ohio

Date of amendment request: November 2, 1995

Description of amendment request: The proposed amendment would revise the Technical Specifications to allow 120 volt AC buses EV-1-A and EV-1-B to be energized from either their normal inverter power supply or from their alternate power supply.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated:

These buses are not used as the initiator of any analyzed accidents. Therefore, the probability of any previously evaluated accident has not increased. If an accident were to occur while the buses are supplied from the alternate power supply, there would