The proposed changes do not affect the operation or response of any plant equipment or introduce any new failure mechanisms. The proposed elimination of the testing requirement line item does not affect the test results since the logic circuitry that processes the safety injection signal and produces a reactor trip will be tested and is tested under functional unit 19 of Table 4.3–1. As such, the changes do not create the possibility of a new or different kind of accident previously evaluated.

3. Involve a significant reduction in the margin of safety.

The proposed changes do not have any adverse impact on the protective boundaries nor do they affect the consequences of any accident analyzed. The operability and surveillance requirements, although relocated to other technical specifications, will still ensure that the system (the radiation monitors) is tested and within limits. The proposed elimination of the testing equipment will not change the performance or operating conditions of the safety systems. The operable reactor trip system instrumentation ensures that the assumptions in the Bases of the Technical Specifications are not affected and ensures that the margin of safety is not reduced. Therefore, the proposed changes do not reduce the margin of safety

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room location:* Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, CT 06360.

Attorney for licensee: Lillian M. Cuoco, Esq., Senior Nuclear Counsel, Northeast Utilities Service Company, P.O. Box 270, Hartford, CT 06141–0270.

*NRC Project Director:* Phillip F. McKee

Pacific Gas and Electric Company, Docket Nos. 50–275 and 50–323, Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, San Luis Obispo County, California

Date of amendment requests: November 14, 1994

Description of amendment requests: The proposed amendment would revise the combined Technical Specifications (TS) for the Diablo Canyon Nuclear Power Plant, Unit Nos. 1 and 2, for the slave relay test frequency from quarterly (Q) to refueling (R). The request would also remove table notation 4 from Table 4.3–2. The associated Bases would also be appropriately revised.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The results of WCAPs 14117 and 13878 demonstrate that slave relays are highly reliable. The WCAPs also provide guidance to assure that slave relays remain highly reliable. The aging assessment concludes that the age/temperature-related degradation of all ND relays, and NE relays produced after May 1990, is sufficiently slow such that a refueling frequency surveillance interval will not significantly increase the probability of slave relay failures. Finally, the evaluation of the interposing slave relays in the emergency diesel generator start circuitry, control room ventilation and auxiliary building ventilation realignments, steam generator blowdown isolation and radwaste isolation systems has concluded that based on the tests of the interposing relays performed during other equipment testing, reasonable assurance is provided that failures will be identified if the associated slave relays are tested on a refueling frequency.

The removal of table notation 4 from TS Table 4.3–2 is an administrative change that eliminates unnecessary redundancy from the TS and does not affect plant operation.

Therefore, the proposed changes do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes do not alter the performance of the ESFAS mitigation systems assumed in the plant safety analysis. Changing the interval for periodically verifying ESFAS slave relays (assuring equipment operability) will not create any new accident initiators or scenarios.

Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated for DCPP.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed changes do not affect the total ESFAS response assumed in the safety analysis since the reliability of the slave relays will not be significantly affected by the increased surveillance frequency.

Therefore, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Local Public Document Room location: California Polytechnic State University, Robert E. Kennedy Library, Government Documents and Maps Department, San Luis Obispo, California 93407

Attorney for licensee: Christopher J. Warner, Esq., Pacific Gas and Electric Company, P.O. Box 7442, San Francisco, California 94120

*NRC Project Director:* William H. Bateman

South Carolina Electric & Gas Company, South Carolina Public Service Authority, Docket No. 50–395, Virgil C. Summer Nuclear Station, Unit No. 1, Fairfield County, South Carolina

*Date of amendment request:* August 18, 1995, as supplemented on November 1, 1995

Description of amendment request: The proposed amendment would revise the Operating License and Technical Specifications to allow for a power uprate to 2900 MWt. The current maximum power level is 2775 MWt.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The probability or consequences of an accident previously evaluated is not significantly increased.

Implementation of uprate power operation does not contribute to any accident evaluated in the FSAR [Final Safety Analysis Report]. The NSSS [Nuclear Steam Supply System] Components (RV [reactor vessel], RCPs [reactor coolant pumps], CRDMs [control rod drive mechanisms], SGs [steam generators], and piping) are compatible with the revised operating conditions. These components have been reanalyzed and the results show that ASME [American Society of Mechanical Engineers] Code requirements remain satisfied and are within the current Licensing Basis.

Interfacing Systems which are important to safety are not adversely impacted and will continue to perform their design function. Overall secondary plant performance is not significantly altered by the proposed changes.

The revision to the Pressure Temperature Limits will not adversely impact the RCS [reactor coolant system] Pressure Boundary. The length of time these curves will be applicable, due to increased neutron fluence, is being reduced. Before the 13 Effective Full Power Years have elapsed, new curves will be generated to reflect the analysis of the specimen capsule and will be derived utilizing NRC approved methodology.

Therefore, since the Reactor Coolant pressure boundary integrity and system functions are not adversely impacted, the probability of occurrence of an accident evaluated in the VCSNS [Virgil C. Summer Nuclear Station] FSAR will be no greater than the original design basis of the plant.

An extensive analysis has been performed to evaluate the consequences of the following accident types currently evaluated in the VCSNS FSAR: