

2) create the possibility of a new or different kind of accident from any accident previously evaluated.

This change is administrative in nature and does not impact plant systems, structures or components. The proposed change will allow the facility's technical specifications to be revised to reflect current containment sphere leakage testing in accordance with Appendix J to 10 CFR Part 50.

3) involve a significant reduction in a margin of safety.

This change is administrative in nature and does not impact plant systems, structures or components.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Project Director: Brian E. Holian, Acting

Georgia Power Company, Oglethorpe Power Corporation, Municipal Electric Authority of Georgia, City of Dalton, Georgia, Docket Nos. 50-424 and 50-425, Vogtle Electric Generating Plant, Units 1 and 2, Burke County, Georgia

Date of amendment request: October 16, 1995

Description of amendment request: Appendix J of 10 CFR Part 50, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," has recently been revised to include Option B. This option allows the implementation of a performance based Type B and C testing program. The proposed change will add a footnote to Technical Specification (TS) 4.6.1.2.d stating that the Type B and C tests scheduled for Unit 1 refueling outage Cycle 6 (1R6) will be conducted in accordance with Option B and using the guidance of Regulatory Guide 1.163, Revision 0. This option is being incorporated into the licensee's request to implement the improved TS. However, the improved TS are not scheduled to become effective until after the Unit 1 refueling outage 1R6.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The change does not involve a significant increase in probability or consequences of an accident previously evaluated. The proposed change does not involve a change to structures, systems, or components which would affect the probability or consequences of an accident previously evaluated in the Vogtle Electric Generating Plant (VEGP) Final Safety Analysis Report (FSAR). The proposed change only provides a mechanism within the Technical Specifications for implementing a performance-based method of determining the frequency for leak rate testing which has been approved by the NRC via a revision to 10 CFR 50, Appendix J.

2. The proposed change will not create the possibility of a new or different kind of accident from any accident previously analyzed. The amendment will not change the design, configuration, or method of plant operation. It only allows for the implementation of Option B of 10 CFR 50, Appendix J for Unit 1 refueling outage 1R6 without violating the plant Technical Specifications.

3. Operation of VEGP, Unit 1, in accordance with the proposed change will not involve a significant reduction in the margin of safety. The proposed change does not affect a safety limit, an LCO [limiting condition for operation], or the way plant equipment is operated. The NRC is aware that changes similar to this proposed change are required in order to implement Option B of 10 CFR 50, Appendix J. In fact, the staff indicates in Paragraph V.B. of Appendix J that Option B or parts thereof may be adopted by a licensee 30 days after the rule becomes effective by submitting notification of its implementing plan and a request for revision to Technical Specifications. Since the NRC has approved the provision for performance-based testing and must approve this Technical Specification[] change before the performance-based Option B can be implemented, the margin of safety will not be significantly reduced.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Burke County Public Library, 412 Fourth Street, Waynesboro, Georgia 30830

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NRC Project Director: Herbert N. Berkow

Gulf States Utilities Company, Cajun Electric Power Cooperative, and Entergy Operations, Inc., Docket No. 50-458, River Bend Station, Unit 1, West Feliciana Parish, Louisiana

Date of amendment request: May 30, 1995 (noticed in the Federal Register July 5, 1995, (60 FR 35080) as supplemented by letter dated November 20, 1995

Description of amendment request: The proposed amendment would revise the Technical Specifications as follows:

1. The Surveillance Frequency for the drywell bypass test is changed from 18 months to 10 years with an increased testing frequency required if performance degrades.

2. The following changes are requested for the drywell air lock testing: (a) the leakage rate surveillance is moved from the air lock Limiting Condition for Operation (LCO) to the drywell LCO, (b) the requirement for the air lock to meet a specific overall leakage limit is deleted, (c) the Note that an inoperable air lock door does not invalidate the previous air lock leakage test is deleted, (d) the Note which required that the air lock leakage test at 3 psid be preceded by pressurizing the air lock to 19.2 psid is moved to the bases, and (e) the Surveillance Frequency for the air lock leakage test and interlock test is changed from 18 months to 24 months.

3. The Actions Notes in the drywell air lock LCO and the drywell isolation valve LCO that identifies that the Actions required by the drywell LCO must be taken when the drywell bypass leakage limit is not met is deleted.

4. The requirement for the drywell air lock seal leakage rate to meet a specific leakage limit is deleted.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration for River Bend Station (RBS) and Grand Gulf Nuclear Station (GGNS), which is presented below:

1. The proposed change does not significantly increase the probability or consequences of an accident previously evaluated.

The requested changes are either administrative changes which clarify the format of the requirement or change the requirement to match the design bases of the plant, a change which relocates the requirement to the Technical Specification Bases, or a change in surveillance interval. Each of these types of change are discussed below:

1. The administrative changes clarify the format of the requirement or change the requirement to match the design bases of the plant. Clarifying administrative format of