refueling interval (24 months + 25% as allowed by Technical Specification 4.0.2).

The proposed change to Surveillance Requirement 4.6.3.2 does not alter the intent or method by which the surveillance is conducted, does not involve any physical changes to the plant, does not alter the way any structure, system, or component functions, and does not modify the manner in which the plant is operated.

Additional assurance of CIV operability is provided by Surveillance Requirement 4.6.3.3. Surveillance Requirement 4.6.3.3 requires the confirmation of the mechanical operability of the CIVs by the inservice inspection program. The proposed change does not modify these requirements.

Equipment performance over the last four operating cycles was evaluated to determine the impact of extending the frequency of Surveillance Requirement 4.6.3.2. This evaluation included a review of surveillance results, preventive maintenance records, and corrective maintenance records. It has been concluded that the CIVs are highly reliable, and that there is no indication that the proposed extension could cause deterioration in valve condition or performance.

As such, the proposed change to the frequency of Surveillance Requirement 4.6.3.2 will not degrade the ability of the CIVs to perform their safety function.

Based on the above, the proposed change to Surveillance Requirement 4.6.3.2 of the Haddam Neck Plant Technical Specifications does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any previously evaluated.

The proposed change to Surveillance Requirement 4.6.3.2 of the Haddam Neck Plant Technical Specifications extends the frequency for verifying that each CIV actuates to its required position in response to a safety injection actuation test signal. The proposal would extend the frequency from at least once per 18 months to at least once per refueling interval (24 months + 25% as allowed by Technical Specification 4.0.2).

The proposed change does not alter the intent or method by which the surveillance is conducted, does not involve any physical changes to the plant, does not alter the way any structure, system, or component functions, and does not modify the manner in which the plant is operated. As such, the proposed change in the frequency of Surveillance Requirement 4.6.3.2 will not degrade the ability of the CIVs to perform their safety function.

Based on the above, the proposed change to Surveillance Requirement 4.6.3.2 of the Haddam Neck Plant Technical Specifications will not create the possibility of a new or different kind of accident from any previously evaluated.

3. Involve a significant reduction in a margin of safety.

The proposed change to Surveillance Requirement 4.6.3.2 of the Haddam Neck Plant Technical Specifications extends the frequency for verifying that each CIV actuates to its required position in response to a safety injection actuation test signal. The proposal would extend the frequency from at least once per 18 months to at least once per refueling interval (24 months + 25% as allowed by Technical Specification Section 4.0.2).

The proposed change does not alter the intent or method by which the surveillance is conducted, does not involve any physical changes to the plant, does not alter the way any structure, system, or component functions, and does not modify the manner in which the plant is operated. As such, the proposed change in the frequency of Surveillance Requirement 4.6.3.2 will not degrade the ability of the CIVs to perform their safety function.

Additional assurance of the operability of the CIVs is provided by Surveillance Requirement 4.6.3.3.

Equipment performance over the last four operating cycles was evaluated to determine the impact of extending the frequency of Surveillance Requirement 4.6.3.2. This evaluation included a review of surveillance results, preventive maintenance records, and corrective maintenance records. It has been concluded that the CIVs are highly reliable, and that there is no indication that the proposed extension could cause deterioration in valve condition or performance.

Based on the above, the proposed change to Surveillance Requirement 4.6.3.2 of the Haddam Neck Plant Technical Specifications does not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Russell Library, 123 Broad Street, Middletown, CT 06457.

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Consumers Power Company, Docket No. 50–155, Big Rock Point Plant, Charlevoix County, Michigan

Date of amendment request: November 8, 1995, as supplemented November 17, 1995

Description of amendment request: The proposed amendment would remove the prescriptive Type A containment leakage test rate frequency of 40 plus or minus 10 months and add a reference to perform containment leakage rate tests in accordance with the criteria specified in Appendix J of 10 CFR Part 50 as amended by approved exemptions. In addition, the proposed amendment would revise the test pressure for Type B and C testing to correct a typographical error.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

Leakage test rate frequency

1) The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

This change is administrative in nature and does not impact plant systems, structures or components. The proposed change will allow the facility's technical specifications to be revised to allow containment sphere leakage testing in accordance with Appendix J to 10 CFR Part 50 as modified by approved exemptions.

2) The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

This change is administrative in nature and does not impact plant syst ems, structures or components. The proposed change will allow the facility's technical specifications to be revised to allow containment sphere leakage testing in accordance with Appendix J to 10 CFR Part 50 as modified by approved exemptions.

3)The proposed change does not involve a significant reduction in a margin of safety.

This change is administrative in nature and does not impact plant systems, structures or components. The underlying purpose of Appendix J is still achieved. Appendix J states that the leakage test requirements provide for periodic verification testing of the leak tightness integrity of the primary reactor containment. The appendix further states that the purpose of the tests is to assure that leakage through the primary containment shall not exceed the allowable leakage rate values as specified in the technical specifications or associated bases. As stated previously, for Big Rock Point and a large percentage of other plants, the Appendix J Type B and C testing programs provide the most significant and meaningful assessment of containment leak tightness. The testing history and structural capability of the containment establish that there is significant assurance that the extended interval between Type A tests will not adversely impact the integrity of the containment.

Test pressure revision

As stated in the technical specification change request, this revision is being performed to be consistent with accident pressure, P_a, used for Big Rock Point. 20 psig is a typographical error. 23 psig has always been used for these tests.

The proposed change does not:
1) involve a significant increase in the probability or consequences of an accident previously evaluated.

This change is administrative in nature and does not impact plant systems, structures or components. The proposed change will allow the facility's technical specifications to be revised to reflect current containment sphere leakage testing in accordance with Appendix J to 10 CFR Part 50.