ACTION: Notice of permits issued under the Antarctic Conservation of 1978, Public Law 95–541.

SUMMARY: The National Science Foundation (NSF) is required to publish notice of permits issued under the Antarctic Conservation Act of 1978. This is the required notice.

FOR FURTHER INFORMATION CONTACT: Robert S. Cunningham or D. Kristen Larsen, Permit Office, Office of Polar Programs, Rm. 755, National Science Foundation, 4201 Wilson Boulevard, Arlington, VA 22230.

SUPPLEMENTARY INFORMATION: On May 30, 1995, and June 29, 1995, the National Science Foundation published a notice in the Federal Register of waste management permit applications received. Permits were issued on September 7, 1995 to the following applicants.

Adventure Network International (ANI), Permit #96WM2-ANI Skip Novak, Permit #96WM3-PELAGIC Robert S. Cunningham, NEPA Compliance Manager, Office of Polar

Programs, National Science Foundation. [FR Doc. 95–24506 Filed 10–2–95; 8:45 am]

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NUCLEAR REGULATORY COMMISSION

[Docket No. 50-440]

The Cleveland Electric Illuminating Company, et al.; Perry Nuclear Power Plant, Unit No. 1; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory
Commission (the Commission) is
considering issuance of several
exemptions from the requirements of 10
CFR Part 50, Appendix J to the
Cleveland Electric Illuminating
Company, Centerior Service Company,
Duquesne Light Company, Ohio Edison
Company, Pennsylvania Power
Company, and Toledo Edison Company
(the licensees), for operation of the Perry
Nuclear Power Plant, Unit 1, located in
Lake County, Ohio.

Environmental Assessment

Identification of the Proposed Action

The proposed action would grant exemptions from the requirements of Sections III.A.5(b)(2), III.B.3, III.C.3, III.A.1(d), III.D.1(a), and III.D.3 of Appendix J to 10 CFR Part 50. Section III.A.5(b)(2) requires that the measured leakage for the containment integrated leak rate test ($L_{\rm am}$) be less than 75% of the maximum allowable leakage rate

 $(0.75~L_a)$. The proposed exemption would permit separate treatment of main steam isolation valve leakage from the containment integrated leak rate tests.

Sections III.B.3 and III.C.3 require that the combined leakage of valves and penetrations subject to Type B and C local leak rate testing be less than 0.6 times the maximum allowable leakage rate $(0.6\ L_a)$. The proposed exemption would permit separate treatment of main steam isolation valve leakage from local leak rate testing.

Section III.A.1(d) requires that all fluid systems that would be open to containment following post-accident conditions, be vented and drained prior to conducting the containment integrated leak rate test. The proposed exemption would permit separate treatment of the main steam line penetrations and would not require them to be vented and drained prior to

conducting containment integrated leak

Section III.D.1.(a) requires that a set of three Type A tests be performed at approximately equal intervals during each 10-year service period and that the third test of each set be conducted when the plant is shut down for the 10-year plant inservice inspection (ISI). The proposed exemption would permit performance of the third Type A test at times other than when the plant is shut down for the 10-year plant ISI.

Section III.D.3 requires that Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years. The proposed exemption would allow the licensee to perform the required Type C tests while the plant is at power.

The proposed action is in accordance with the licensee's application for exemption dated October 21, 1994.

The Need for the Proposed Action

Assumptions used in both the Perry FSAR and Standard Review Plan 15.6.5, Appendix D, "Radiological Consequences of a Design Basis Loss-of-Coolant Accident," for computing the total radiological consequences from a hypothetical loss-of-coolant accident (LOCA), include separate contributions for the containment leak rate and the main steam line isolation valve leak rate. The value for the maximum allowable containment leak rate, La, of 0.2%/day, was established based on separate accounting for the main steam line isolation valve leak rate. The proposed exemption from Section III.A.5 (b)(2) is needed to allow separate treatment of main steam line isolation valve leakage from the containment integrated leak rate.

Sections III.B.3 and III.C.3 of Appendix J state that the combined leakage from all valves and penetrations subject to Type B and C local leak rate testing shall be less than 0.6 $L_{\rm a}$. However, separate leakage limits have been established for the main steam isolation valves at Perry. An exemption from Sections III.B.3 and III.C.3 is needed to allow separate treatment of main steam isolation valve leakage from local leak rate testing.

Section III.A.1(d) requires that those systems that would be exposed to the containment atmosphere following a design basis LOCA, be vented and drained prior to conducting the containment integrated leak rate test. However, the main steam piping between the inboard and outboard isolation valves at Perry are filled with water during the containment integrated leak rate tests. This practice ensures that any leakage through the isolation valves will not contribute to the overall containment test results. An exemption from Section III.A.1(d) is needed to allow this alternative practice.

The proposed exemption from 10 CFR Part, Appendix J, Section III.D.1(a), is needed to avoid unnecessary restraints in outage scheduling. The licensee proposed to perform the three Type A tests at approximately equal intervals within each 10-year period, with the third test of each set conducted as close as practical to the end of the 10-year period. However, there would be no required connection between the Appendix J 10-year interval and the ISI 10-year interval.

Section III.D.3 of Appendix J to 10 CFR part 50 states that Type C tests shall be performed during each reactor shutdown for refueling but in no case at intervals greater than 2 years. The proposed exemption is needed to allow the option to perform Type C testing at power.

Environmental Impacts of the Proposed Action

The Commission has completed its evaluation of the proposed action and concludes that the exemption would not significantly increase the probability or amount of expected primary containment leakage, and that containment integrity would thus be maintained.

The change will not increase the probability or consequences of accidents, no changes are being made in the types of any effluents that may be released offsite, and there is no significant increase in the allowable individual or cumulative occupational radiation exposure. Accordingly, the Commission concludes that there are no