licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated. The current analysis requires the SLC system to be capable of bringing the reactor 3% delta - k subcritical assuming a cold xenon free condition. The increase in SLC storage tank boron concentration limits will ensure this capability is maintained for future reload cores using the same 3% delta - k shutdown reactivity margin without imposing restrictions in cycle exposure for current and future anticipated core configurations. The change in the surveillance frequency for SLC pump operability testing to once each three months is in agreement with the ASME Code. The relaxation of the testing interval for the SLC pumps decreases pump degradation, and eliminates an unnecessary burden on personnel resources without compromising plant safety. In addition, the administrative changes only correct typographical and editorial errors.

Since these proposed changes do not affect precursors for any accident or transient analyzed in Chapter 14 of the USAR, there is no increase in the probability of any accident previously evaluated. Furthermore, since these changes will ensure the ability of the SLC system to mitigate the consequences of an accident for future anticipated core designs, they do not involve a significant increase in the consequences of any accident previously evaluated.

2. The proposed changes will not create the possibility of a new or different kind of accident from any accident previously evaluated. The change in the SLC storage tank boron concentration limits will ensure that a cold xenon-free reload core can be brought to a subcritical condition as previously analyzed. The change in the frequency of the SLC pump operability testing to once each three months is in agreement with the ASME Code. The relaxation in the testing interval for the SLC pumps decreases pump degradation, and eliminates an unnecessary burden on personnel resources without compromising plant safety. In addition, the administrative changes only correct typographical and editorial errors.

These proposed changes do not affect the design, function, or operation of the SLC or any other system. Also, these changes do not introduce any new modes of operation or modify existing equipment design. Therefore, they do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed changes will not create a significant reduction in the margin of safety. The proposed increase in the required boron concentration in the reactor pressure vessel will ensure the SLC system will be capable of bringing a cold xenon-free reload core subcritical while maintaining the 3% delta - k shutdown reactivity margin as specified in the previous operating cycle. The change in the frequency of SLC pump operability testing to once each three months is in

agreement with the ASME Code. The relaxation in the testing interval for the SLC pumps decreases pump degradation, and eliminates an unnecessary burden on personnel resources without compromising plant safety. In fact, it increases SLC system availability. In addition, the administrative changes only correct typographical and editorial errors. Therefore, it is concluded that the requested changes do not create a significant reduction in the existing margin of safety as defined in the Technical Specifications.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Auburn Public Library, 118 15th Street, Auburn, Nebraska 68305

Attorney for licensee: Mr. John R. McPhail, Nebraska Public Power District, Post Office Box 499, Columbus, Nebraska 68602-0499

*NRC Project Director:* William D. Beckner North Atlantic Energy Service Corporation, Docket No. 50-443, Seabrook Station, Unit No. 1, Rockingham County, New Hampshire

Date of amendment request: June 16, 1995

Description of amendment request: The proposed amendment would change the minimum boron concentration specified for the refueling water storage tank (RWST) in Limiting Condition for Operation (LCO) in Technical Specification (TS) 3.1.2.5 and would replace the minimum specified concentration for boron with an acceptable range of boron concentration for the RWST and the accumulators in the LCOs for TS 3.1.2.6, 3.5.1.1, and 3.5.4.

Basis for proposed no significant hazards consideration determination:As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The NRC staff's review is presented below.

A. The changes do not involve a significant increase in the probability or consequences of an accident previously evaluated (10 CFR 50.92(c)(1)) because the changes are proposed to assure that the post-event shutdown margin required by the Technical Specifications will continue to be met and the consequences of a boron dilution event will remain as previously evaluated. The changes do not affect the design or manner of operation of any structure,

system, or component important to safety.

B. The changes do not create the possibility of a new or different kind of accident from any accident previously evaluated (10 CFR 50.92(c)(2)) because they do not affect the manner by which the facility is operated and do not involve a change to any structure, system, or component important to safety. The proposed changes merely assure that station will be operated within original design limits.

C. The changes do not involve a significant reduction in a margin of safety (10 CFR 50.92(c)(3)) because the proposed changes merely assure that the station will continue to be operated within the original design limits. Therefore, the acceptance criteria for previously evaluated accidents will continue to be met.

Based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Exeter Public Library, Founders Park, Exeter, NH 03833.

Attorney for licensee: Thomas Dignan, Esquire, Ropes & Gray, One International Place, Boston MA 02110-2624.

*NRC Project Director:* Phillip F. McKee

## Northeast Nuclear Energy Company (NNECO), Docket No. 50-245, Millstone Nuclear Power Station, Unit 1, New London County, Connecticut

Date of amendment request: July 11, 1995

Description of amendment request: The proposed amendment modifies Technical Specification 3.5.F.7 to also allow the use of pull-to-lock switches to defeat the automatic initiation of the emergency core cooling system (ECCS) while in the refuel condition. The proposed amendment also makes administrative changes and makes changes to the associated Bases section.

Basis for proposed no significant hazards consideration determination:As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

NNECO has reviewed the proposed change in accordance with 10 CFR 50.92 and concluded that the change does not involve a significant hazards consideration (SHC). The basis for this conclusion is that the three criteria of 10 CFR 50.92(c) are not compromised. The proposed change does not