

expertise in specific key plant functions. The Millstone Station has one SORC. The SORC is also an oversight group whose charter is to advise the Senior Vice President - Millstone Station on all matters related to nuclear safety at the Millstone site. The Haddam Neck Plant, being a single unit site, has one PORC, which advises the Vice President - Haddam Neck Plant. The members of the Haddam Neck Plant PORC will be selected by the Vice President - Haddam Neck Plant based on their knowledge and expertise in specific key plant functions. The PORC and SORC add to the defense-in-depth concept provided by the design, operation, maintenance, and quality oversight by promoting excellence through the conduct of their affairs and by maintaining a diligent watch over their responsibilities.

These administrative changes will revise the composition section of the technical specifications for the PORC members. Millstone Unit individuals will be appointed by the Nuclear Unit Directors if the individual meets one or more of the following areas of expertise: Plant Operations, Engineering, Reactor Engineering, Maintenance, Instrumentation and Controls, Health Physics, Chemistry, Work Planning and Control, and Quality Services. The Haddam Neck Plant, due to its broader scope of review also include[s] an individual experienced in Security and specific experience in Electrical Maintenance and Mechanical Maintenance. The individuals who will serve on PORC shall continue to meet the criteria of ANSI N18.1-1971. This approach is consistent with the standard technical specifications and NUREG 0800, Section 13.4. For SORC at the Millstone Station, the method of identifying who shall serve as Vice Chairperson has been modified for clarity. The Site Services Director position is proposed to be eliminated since this position no longer exists. The functions previously performed by this individual have been assumed by those individuals who currently serve on SORC. Finally, [the TS relating to] the individual who shall represent Quality and Assessment Services shall be modified to allow a qualified member of Quality and Assessment Services to serve on SORC.

The remaining portions of the technical specifications related to PORC and SORC are not being revised.

These modifications broaden the unit committee participation and reflect current organizational positions and will not increase the probability of occurrence or the consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed administrative enhancements to the composition of the PORC and Millstone Station SORC will not affect the way in which the units are physically operated. These administrative changes to PORC and SORC continue to meet the guidelines of ANSI N18.7-1976. The modifications to PORC and SORC continue to allow these groups to provide a thorough review of activities at the units.

The proposed modification does not impact any initiating events, and, therefore, cannot create the possibility of any new or different kind of accident from any accident previously evaluated.

3. Involve a significant reduction in a margin of safety.

These proposed administrative changes will not impact the margin of safety provided by PORC and SORC. The PORC and SORC will continue to be staffed by qualified individuals experienced in the operation of the plants. These administrative changes will modify how the composition of the PORC and SORC members are presented in the technical specifications, but will not adversely impact their ability to review and comment on operations at the units.

These changes do not impact any protective boundaries nor do they impact the safety limits for the protective boundaries. These proposed changes are administrative in nature. Therefore, there is no reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Russell Library, 123 Broad Street Middletown, Connecticut 06457, for the Haddam Neck Plant, and the Learning Resources Center, Three Rivers Community-Technical College, 574 New London Turnpike, Norwich, CT 06360, for Millstone 1, 2, and 3.

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Consumers Power Company, Docket No. 50-255, Palisades Plant, Van Buren County, Michigan

Date of amendment request: July 5, 1995

Description of amendment request: The proposed amendment would change the Administrative Controls section of the Palisades Technical Specifications. The changes involve deleting training requirements in the Administrative Controls section, revising the Plant Review Committee composition, and revising the function and composition of the plant safety and licensing staff review requirements.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

A. Involve a significant increase in the probability or consequences of an accident previously evaluated.

This change does not affect the probability or consequences of an accident. The changes are administrative, deleting an unnecessary specification on staff training requirements, eliminating the specific references to the Nuclear Engineering and Construction Organization (NECO) staff, and requiring that the Plant Review Committee (PRC) chairman, alternate chairman, and members be designated in administrative procedures by the Plant General Manager. Further administrative changes clarify the function of the Plant Safety and Licensing organization and eliminate the numerical requirement for five staff members to fulfill the organization function.

The removal of an obsolete staff training requirement does not diminish the regulatory requirement to have an adequately trained staff. The accredited training programs for the plant staff ensure an appropriate level of training is conducted to maintain an appropriate skill and knowledge base for the staff. The requirements of 10CFR55 provide the necessary rules for operator licenses. Since a trained staff will be maintained, there will [be] no increase in the probability or consequences of an accident as a result of this change.

The composition of the PRC will not be affected by this change as it will, at a minimum, be comprised of personnel from the operations, engineering, radiological services and maintenance departments as required by the Technical Specifications. The composition of the Plant Safety and Licensing organization as a whole may change. The function of the organization as it relates to these Technical Specifications, however, will not be affected. These changes have no effect on the plant accident analyses. Qualified personnel will continue to conduct the PRC and Plant Safety and Licensing reviews. Therefore, the changes do not increase the probability or consequences of an accident.

B. Create the possibility of a new or different kind of accident from any previously evaluated.

The proposed changes are administrative and do not create the possibility of a new or different kind of accident. Staff training will continue to meet the accreditation requirements of the National Academy for Nuclear Training Accreditation Board and the requirements for the Systematic Approach to Training. Operators' license training will continue to meet the regulatory requirements of 10CFR55. Activities conducted by the Plant Review Committee and the Plant Safety and Licensing staff will continue to be accomplished by a staff which meets the qualification requirements of the Technical Specifications. These administrative changes will not affect the operation of the plant or the safety function of plant equipment nor will it affect the quality of the review activities. Therefore, there will be no possibility that a new or different kind of accident will be created.

C. Involve a significant reduction in a margin of safety.

The changes do not affect installed plant equipment nor do they affect plant