

following a postulated loss-of-coolant accident (LOCA) by—

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16. In § 50.46, paragraph (a)(1)(i) is revised to read as follows:

§ 50.46 Acceptance criteria for emergency core cooling systems for light water nuclear power reactors.

(a)(1)(i) Each boiling or pressurized light-water nuclear power reactor fueled with uranium oxide pellets within cylindrical zircaloy or ZIRLO cladding, other than a reactor facility for which the certifications required under § 50.82(a)(1) have been submitted, must be provided with an emergency core cooling system (ECCS) that must be designed so that its calculated cooling performance following postulated loss-of-coolant accidents conforms to the criteria set forth in paragraph (b) of this section. ECCS cooling performance must be calculated in accordance with an acceptable evaluation model and must be calculated for a number of postulated loss-of-coolant accidents of different sizes, locations, and other properties sufficient to provide assurance that the most severe postulated loss-of-coolant accidents are calculated. Except as provided in paragraph (a)(1)(ii) of this section, the evaluation model must include sufficient supporting justification to show that the analytical technique realistically describes the behavior of the reactor system during a loss-of-coolant accident. Comparisons to applicable experimental data must be made and uncertainties in the analysis method and inputs must be identified and assessed so that the uncertainty in the calculated results can be estimated. This uncertainty must be accounted for, so that, when the calculated ECCS cooling performance is compared to the criteria set forth in paragraph (b) of this section, there is a high level of probability that the criteria would not be exceeded. Appendix K to this part, Part II Required Documentation, sets forth the documentation requirements for each evaluation model.

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17. In § 50.48, paragraph (f) is added to read as follows:

§ 50.48 Fire protection.

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(f) Licensees that have submitted the certifications required under § 50.82(a)(1) shall maintain a fire protection program to address the potential for fires which could cause the release or spread of radioactive materials (i.e., which could result in a radiological hazard).

(1) The objectives of the fire protection program are to:

(i) Reasonably prevent such fires from occurring;

(ii) Rapidly detect, control, and extinguish those fires which do occur and which could result in a radiological hazard; and

(iii) Ensure that the risk of fire-induced radiological hazards to the public, environment and plant personnel is minimized.

(2) The fire protection program must be assessed by the licensee on a regular basis and revised as appropriate throughout the various stages of facility decommissioning.

(3) The licensee may make changes to the fire protection program without NRC approval if these changes do not reduce the effectiveness of fire protection for facilities, systems and equipment which could result in a radiological hazard, taking into account the decommissioning plant conditions and activities.

18. In § 50.49, paragraph (a) is revised to read as follows:

§ 50.49 Environmental qualification of electric equipment important to safety for nuclear power plants.

(a) Each holder of or an applicant for a license for a nuclear power plant, other than a reactor facility for which the certifications required under § 50.82(a)(1) have been submitted, shall establish a program for qualifying the electric equipment defined in paragraph (b) of this section.

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19. In § 50.51, the section heading is revised, the existing paragraph is designated paragraph (a), and paragraph (b) is added to read as follows:

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§ 50.51 Continuation of license.

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(b) Each license will continue in effect beyond the expiration date, if necessary, with respect to possession of the production or utilization facility, until the Commission notifies the licensee in writing that the license is terminated. During any period of continued effectiveness of a license beyond the license's stated expiration date, except for a license which is in timely renewal status under § 2.109 of this chapter, the licensee is prohibited from operating the production or utilization facility and shall—

(1) Take actions necessary to decommission and decontaminate the facility and continue to maintain the facility, including the storage, control and maintenance of the spent fuel, in a safe condition, and

(2) Conduct activities in accordance with all other restrictions applicable to

the facility in accordance with the NRC regulations and the provisions of the specific part 50 license for the facility.

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20. In § 50.54, paragraphs (o) and (y) are revised to read as follows:

§ 50.54 Conditions of licenses.

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(o) Primary reactor containments for water cooled power reactors, other than reactor facilities for which the certifications required under § 50.82(a)(1) have been submitted, shall be subject to the requirements set forth in Appendix J to this part.

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(y) Licensee action permitted by paragraph (x) of this section shall be approved, as a minimum, by a licensed senior operator, or, at a nuclear power reactor for which the certifications required under § 50.82(a)(1) have been submitted, by either a licensed senior operator or a certified fuel handler, prior to taking the action.

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21. In § 50.59, paragraphs (d), (e), and (f) are added to read as follows:

§ 50.59 Changes, tests and experiments.

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(d) All the provisions of this section shall apply to each nuclear power reactor licensee that has submitted the certification of permanent cessation of operations required under § 50.82(a)(1).

(e) (1) A nuclear power reactor licensee that has submitted the certification of permanent cessation of operations required under § 50.82(a)(1) may conduct activities with regard to the facility, subject to the limitations described in paragraph (a) of this section, provided the changes would not:

(i) Foreclose the release of the site for possible unrestricted use,

(ii) Significantly increase decommissioning costs,

(iii) Cause any significant environmental impact not previously reviewed, or

(iv) Violate the terms of the licensee's existing license.

(2) For changes not meeting any of the criteria in this paragraph or paragraph (a) of this section, the licensee shall submit an application for amendment pursuant to § 50.90.

(f) The provisions of paragraphs (a) through (c) of this section apply to each non-power reactor licensee whose license no longer authorizes operation of the reactor.

22. In § 50.60, paragraph (a) is revised to read as follows: