

requirements or safety limits. The Technical Specifications will continue to require operations within analyzed core operating limits and appropriate actions be taken when, or if, limits are exceeded. There will be no changes to the physical design of the plant as a result of adding the proposed references to Section 6.9.3.2. The results of analytical determination of core operating limitations is not assumed as the initiator of any analyzed event, and the approved safety analysis is still applicable. Therefore, the proposed amendment to Technical Specification 6.9.3.2 does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed amendment does not remove or modify existing Technical Specification requirements or safety limits. The Technical Specifications will continue to require operation within analyzed core operating limits and appropriate actions be taken when, or if, limits are exceeded. The technical methodology outlined in the three new reports is in accordance with the accepted principals, and the specific reports proposed for inclusion in the Technical Specifications by this request have been previously approved by NRC for use at WNP-2 as a basis for core reload analyses. Therefore, the proposed amendment to Technical Specification 6.9.3.2 does not create the possibility of a new or different type of accident from any accident previously evaluated.

3. The proposed change does not involve a significant reduction in a margin of safety.

Plant safety limits are established through LCOs, limiting safety systems settings, and safety limits specified in the Technical Specifications. There will be no changes to either the physical design of the plant or to any of these settings and limits as a result of adding the proposed references to Section 6.9.3.2. The ability to mitigate the consequences of all accidents previously evaluated will be maintained and nuclear safety is not adversely affected because the technical methodology outlined in the three new reports is in accordance with accepted principals, and the specific reports proposed for inclusion in the Technical Specifications by this request have been previously approved by NRC for use at WNP-2 as a basis for core reload analyses. Therefore, the proposed amendment to Technical Specification 6.9.3.2 does not significantly reduce any margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room*  
location: Richland Public Library, 955 Northgate Street, Richland, Washington 99352

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*NRC Project Director:* William H. Bateman

**Washington Public Power Supply System, Docket No. 50-397, Nuclear Project No. 2, Benton County, Washington**

*Date of amendment request:* June 6, 1995

*Description of amendment request:* The proposed amendment would change the Index of the WNP-2 technical specifications by deleting reference to the Bases pages. Consistent with the requirements of 10 CFR 50.36(a), which states that the Bases shall not become part of the technical specifications, the Bases information will be consolidated into a controlled plant document.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a) the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes are administrative and do not remove or modify existing Technical Specification requirements or safety limits. There will be no changes to the physical design of the plant as a result of the proposed change. The Bases information, per 10 CFR 50.36(a), is not part of the Technical Specifications and will be consolidated into a controlled plant document. Future changes to the Bases will be evaluated per 10 CFR 50.59. Therefore, the proposed changes to the Technical Specification Index do not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes are administrative and do not remove or modify existing Technical Specification requirements or safety limits. There will be no changes to the physical design of the plant or alteration of any operational practice as a result of the proposed change. The Bases information, per 10 CFR 50.36(a), is not part of the Technical Specifications and will be consolidated into a controlled document. Future changes to the Bases will be evaluated under 10 CFR 50.59. Therefore, the proposed changes to the Technical Specifications Index do not create the possibility of a new or different type of accident.

3. The proposed change does not involve a significant reduction in a margin of safety.

Plant safety limits are established through LCOs, limiting safety system settings, and

safety limits specified in the Technical Specifications. There will be no changes to either the physical design of the plant or to any of these settings and limits as a result of modifying the Technical Specification Index. The ability to mitigate the consequences of all accidents previously evaluated will be maintained and nuclear safety is not impacted. The Bases information, per 10 CFR 50.36(a), is not part of the Technical Specifications and will be consolidated into a controlled document. Future changes to the Bases will be evaluated under 10 CFR 50.59. Therefore, the proposed amendment does not significantly reduce any margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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*NRC Project Director:* William H. Bateman

**Washington Public Power Supply System, Docket No. 50-397, Nuclear Project No. 2, Benton County, Washington**

*Date of amendment request:* June 6, 1995

*Description of amendment request:* The proposed amendment would change Technical Specification (TS) 6.0, "Administrative Controls" for WNP-2. Specifically, the changes would (a) reflect Supply System titles for senior management throughout TS 6.0, (b) modify the Plant Operations Committee (POC) composition to specify members according to functional areas rather than by organizational titles (c) replace the Plant Manager as the POC Chairman with an individual appointed by the Plant General Manager, and (d) make an editorial correction.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a) the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The senior management title changes are title changes only and will not impact the plant safety responsibilities associated with these positions. The removal of the Plant Operations Committee (POC) organizational