

that it does not represent a significant hazards consideration based on criteria established in 10 CFR 50.92(c). Operation of Sequoyah Nuclear Plant (SQN) in accordance with the proposed amendment will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change to TS 3.9.4, Containment Building Penetrations, would allow the containment personnel airlocks (PALs) to be open during fuel movement and core alteration. The PALs are not an initiator to any accident. The position of the PAL doors (open or closed) during fuel movement and core alterations has no effect on the probability of any accident previously evaluated.

All doses from a fuel handling accident (FHA) for the proposed change remain well below the 10 CFR 100 limits. The proposed change will reduce the dose to workers inside containment in the event of a FHA by allowing more rapid egress from containment. The wear on the PAL doors will significantly be decreased; therefore, increasing the reliability of the PAL doors in the event of an accident.

Since the probability of a FHA is not affected by the airlock door positions, and the doses remain within acceptable limits, the proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any previously analyzed.

As previously stated, the PAL doors are not accident initiators. The open PAL doors do not represent a significant change in the configuration of the plant; therefore, does not create a new or different type of accident from any previously analyzed.

3. Involve a significant reduction in a margin of safety.

The margin of safety provided for an FHA inside containment remains well below the 10 CFR 100 limits. Therefore, this proposed change to allow the PAL doors to remain open during fuel movement or core alterations does not involve a significant reduction in the margin of safety.

The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room
location: Chattanooga-Hamilton County Library, 1101 Broad Street, Chattanooga, Tennessee 37402

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 11H, Knoxville, Tennessee 37902

NRC Project Director: Frederick J. Hebdon

Washington Public Power Supply System, Docket No. 50-397, Nuclear Project No. 2, Benton County, Washington

Date of amendment request: September 2, 1992

Description of amendment request: The proposed amendment revises the surveillance criteria for the source range monitors (SRMs) to incorporate a more conservative signal-to-noise (S/N) ratio, as recommended by General Electric for this system.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a) the licensee has provided its analysis of the issue of no significant hazards consideration. The NRC staff has reviewed the licensee's analysis against the standards of 10 CFR 50.92(c). The NRC staff's review is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The SRM instrumentation is not assumed to be an initiator of any analyzed event. The SRM instrumentation provides monitoring of neutron flux levels to give the control room operator early indication of unexpected subcritical multiplication that could be indicative of an approach to criticality. As such, action could be taken on the indication to avert or minimize the consequences of the event. However, the SRM function is not relied upon in any design bases or transient analysis. Rod motion interlocks and other instrumentation are relied on in the accident analysis to avert an accident. The change in acceptable count rate and signal-to-noise ratio preserves the confidence level of the General Electric design. As a result, the consequences of any analyzed events are unaffected because the change does not alter any system or component design assumptions or operation. Therefore, no significant increase in the probability or consequences of an accident previously evaluated will be involved.

2. The proposed change does not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed change in SRM count rate and S/N ratio values does not change modes of plant operation or require physical modifications. The WNP-2 design basis accident and transient analyses do not rely on the SRMs to assume plant safety. Therefore, the proposed change does not create the

possibility of a new or different kind of accident.

3. The proposed change does not involve a significant reduction in a margin of safety.

The proposed change does not involve a significant reduction in a margin of safety. The design basis to assure SRM operability is based on an instrument count rate that will assure the SRMs will provide early indication of subcritical multiplication with a 95-percent confidence level. Requiring the count rate to be greater than or equal to 0.7 counts per second (cps) with a S/N ratio greater than or equal to 20, or greater than or equal to 3 cps with a S/N ratio greater than or equal to 2 (vs. a count rate of greater than or equal to 0.5 cps with a S/N ratio greater than or equal to 2 in current TS) ensures the design 95-percent confidence level is maintained when verifying SRM operability. Therefore, the margin of safety is not affected by this change.

Based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room
location: Richland Public Library, 955 Northgate Street, Richland, Washington 99352

Attorney for licensee: M. H. Philips, Jr., Esq., Winston & Strawn, 1400 L Street, N.W., Washington, D.C. 20005-3502

NRC Project Director: William H. Bateman

Washington Public Power Supply System, Docket No. 50-397, Nuclear Project No. 2, Benton County, Washington

Date of amendment request: June 6, 1995

Description of amendment request: The proposed amendment would change Technical Specification 6.9.3.2. The change would add references to three topical reports describing analytical methods that may be used in determining reactor core operating limits for reload licensing applications.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a) the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed amendment does not remove or modify existing Technical Specification