Facility Operating License No. NPF-63: Amendment revises the Technical Specifications.

Date of initial notice in Federal Register: September 28, 1994 (59 FR 49431). The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated December 29, 1994.

No significant hazards consideration comments received: No.

Local Public Document Room location: Reference and Documents Department, Penfield Library, State University of New York, Oswego, New York 13126.

Northeast Nuclear Energy Company, et al., Docket No. 50–336, Millstone Nuclear Power Station, Unit No. 2, New London County, Connecticut

Date of application for amendment: April 22, 1994.

Brief description of amendment: The amendment changes Table 3.3–9 of the Technical Specifications by modifying the indicated measurement range for the neutron flux monitor on the remote shutdown panel. The amendment also includes some corrections of typographical errors in the Technical Specifications.

Date of issuance: December 20, 1994. Effective date: As of the date of issuance to be implemented within 30 days.

Amendment No.: 183.

Facility Operating License No. DPR-65. Amendment revised the Technical Specifications.

Date of initial notice in **Federal Register**: May 25, 1994 (59 FR 27059).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated December 20, 1994.

No significant hazards consideration comments received: No.

Local Public Document Room location: Learning Resources Center, Thames Valley State Technical College, 574 New London Turnpike, Norwich, Connecticut 06360.

Northeast Nuclear Energy Company, et al., Docket No. 50–423, Millstone Nuclear Power Station, Unit No. 3, New London County, Connecticut

Date of application for amendment: July 22, 1994.

Brief description of amendment: The amendment (1) changes the title of Figure 3.1–5 to be consistent with the applicable Limiting Condition For Operation (LCO), (2) relocates the Chemical and Volume Control System (CVCS) valve position requirements to the Reactivity Control Systems— Shutdown Margin specifications, and

(3) consolidates action statements to be expressed in the LCOs rather than in Surveillance Requirements. The amendment also clarifies the requirements for calculating the heat flux hot channel factor $F_Q(z)$ when using the base load option.

Date of issuance: December 29, 1994. Effective date: As of the date of issuance to be implemented within 30 days

. Amendment No.: 99.

Facility Operating License No. NPF-49. Amendment revised the Technical Specifications.

Date of initial notice in **Federal Register**: August 31, 1994 (59 FR 45029)

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated December 29, 1994.

No significant hazards consideration comments received: No.

Local Public Document Room location: Learning Resources Center, Three Rivers Community-Technical College, Thames Valley Campus, 574 New London Turnpike, Norwich, CT 06360.

Northeast Nuclear Energy Company, et al., Docket No. 50–423, Millstone Nuclear Power Station, Unit No. 3, New London County, Connecticut

Date of application for amendment: June 2, 1994, as supplemented August 25, 1994.

Brief description of amendment: The amendment revises the Technical Specifications (TS) to remove expired one-time extensions of surveillance, removes an obsolete definition of charging pump operability, and incorporates 11 line item improvements in accordance with the guidance provided in Generic Letter 93–05. Several editorial changes have been made to renumber TS pages and delete the blank pages from the TS.

Date of issuance: January 3, 1995. Effective date: As of the date of issuance to be implemented within 30 days.

Amendment No.: 100.

Facility Operating License No. NPF-49. Amendment revised the Technical Specifications.

Date of initial notice in Federal Register: September 14, 1994, (59 FR 47170).

The Commission's related evaluation of the amendment is contained in a Safety Evaluation dated January 3, 1995.

No significant hazards consideration comments received: No.

Local Public Document Room location: Learning Resources Center, Three Rivers Community-Technical College, Thames Valley Campus, 574 New London Turnpike, Norwich, CT 06360.

Northern States Power Company, Docket Nos. 50-282 and 50-306, Prairie Island Nuclear Generating Plant, Unit Nos. 1 and 2, Goodhue County, Minnesota

Date of application for amendments: October 3, 1994, as supplemented November 30, 1994.

Brief description of amendments: The amendments revise Prairie Island **Nuclear Generating Plant Technical** Specification 4.6, "Periodic Testing of Emergency Power Systems.' Specifically, the amendments modify the emergency diesel generator (EDG) 24-hour load test requirements to provide an indicated load range of 103-110 percent of the continuous rating. These amendments also rephrase various EDG test requirements to provide clarity and delete the requirements to verify that the autoconnected loads do not exceed 3000 kilowatts (Unit 2 5100 kilowatts).

Date of issuance: January 5, 1995. Effective date: January 5, 1995, with full implementation within 30 days.

Amendment Nos.: 113 and 106. Facility Operating License Nos. DPR-42 and DPR-60. Amendments revised the Technical Specifications.

Date of initial notice in Federal Register: November 9, 1994 (59 FR 55877).

The November 30, 1994, request provided additional clarification that was within the scope of the initial notice and did not affect the staff's proposed no significant hazards consideration findings.

The Commission's related evaluation of the amendments is contained in a Safety Evaluation dated January 5, 1995.

No significant hazards consideration comments received: No.

Local Public Document Room location: Minneapolis Public Library, Technology and Science Department, 300 Nicollet Mall, Minneapolis, Minnesota 55401.

Omaha Public Power District, Docket No. 50-285, Fort Calhoun Station, Unit No. 1, Washington County, Nebraska

Date of amendment request: October 7, 1994.

Brief description of amendment: The amendment (1) deletes the surveillance requirements contained in Technical Specification (TS) 3.6(3)a for the raw water backup valves to the containment cooling coils, (2) deletes the surveillance requirements in TS 3.2, Table 3–5, item 6, for raw water valves, and (3) revises the basis of TS 2.4 to reflect these changes.