The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment, will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed changes to extend the calibration frequency do not represent a physical change to the plant as described in the NMP1 Final Safety Analysis Report (Updated). However, this change results in increasing the analytical limits for the APRM flow based scram and rod block by 2% and 8% respectively. The proposed changes do not alter the plant configuration and the initial conditions used for the design basis accident analysis are still valid. Thus, no potential initiating events are created which would cause any new or different kinds of accidents. As such, the plant initial conditions utilized for the design basis accident analysis are still valid. Therefore, operation of Nine Mile Point Unit 1 in accordance with the proposed change will not create the possibility of a new or different kind of accident from any previously assessed.

The operation of Nine Mile Point Unit 1, in accordance with the proposed amendment, will not involve a significant reduction in a margin of safety.

The analytical limits for the APRM flow biased scram and rod block increase by 2% and 8% respectively. The trip units in the APRM and recirculation flow instrumentation systems will continue to be calibrated every three months. In addition, the entire APRM and recirculation flow instrumentation systems will still be subject to Instrument Channel Tests every three months. These tests, together with the calibration of the flow square rooters and summers once per year and the flow transmitters once per operating cycle, will assure that system reliability and availability are maintained at their current levels. Reanalysis of the design basis transients was performed utilizing these new values. The results showed that the increase had an insignificant effect on the consequences of these events. Therefore, the proposed amendment will not involve a significant reduction in the margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Reference and Documents Department, Penfield Library, State University of New York, Oswego, New York 13126.

Attorney for licensee: Mark J. Wetterhahn, Esquire, Winston & Strawn, 1400 L Street, NW., Washington, DC 20005–3502.

NRC Project Director: Michael J. Case

Niagara Mohawk Power Corporation, Docket No. 50–410, Nine Mile Point Nuclear Station, Unit 2, Oswego County, New York

Date of amendment request: December 13, 1994.

Description of amendment request: The proposed amendment would revise Technical Specification Table 3.6.1.2–1, ''Allowable Leak Rates Through Valves in Potential Bypass Leakage Paths,'' to increase the maximum allowable leakage rate of each of the eight main steamline isolation valves from 6.0 scfh to 24.0 scfh.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

The operation of Nine Mile Point Unit 2, in accordance with the proposed amendment, will not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes to Technical Specification Table 3.6.1.2–1 would allow a maximum leakage of 24.0 scfh for each of the eight MSIVs [main steamline isolation valves]. The current Technical Specifications allow a maximum leakage for an MSIV of 6.0 scfh.

Closure of one or more of the MSIVs at rated power is a pressure transient for the reactor coolant pressure boundary. This pressure transient is evaluated in Section 15.2.4 of the USAR [Updated Safety Analysis Report]. Closure of MSIV(s), as analyzed in the USAR, could occur due to manual or automatic actions. A change to the leakage limit for the MSIVs does not affect either the manual or automatic actions that would close the MSIVs. Therefore, the proposed change to the table cannot affect the probability of the closure of one or more MSIVs at rated power.

The radiological evaluation of the DBA–LOCA [Design Basis Accident—Loss-of-Coolant Accident] incorporates a maximum leakage of 24.0 scfh for each of the four main steam lines. In addition, the revised radiological evaluation includes the impact of the proposed license amendment currently under review by the Staff which would increase the rated operation of NMP2 from 3323 to 3467 megawatts thermal (see NMPC letter dated July 22, 1993 to the NRC). The revised radiological evaluation also includes the impact of License Amendment No. 56 (see NMPC letter dated July 1, 1994 to the NRC and License Amendment No. 56, dated August 30, 1994).

The new doses from the revised radiological analysis for a DBA–LOCA, as shown in Table 1 [of December 13, 1994, amendment request], continue to remain below 10 CFR [Part] 100 guideline values and GDC [General Design Criterion] 19 limits. The impact of the increased MSIV leakage on vital area access and equipment qualification is minimal and acceptable. Therefore, operation with the proposed change to the Technical Specifications will not significantly increase the consequences of an accident previously evaluated.

The operation of Nine Mile Point Unit 2, in accordance with the proposed amendment, will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The safety function of the MSIVs is to isolate the main steam lines in a timely manner to preclude the uncontrolled leakage of radioactive steam. This is accomplished by providing the MSIVs with the capability of rapidly closing automatically in response to various plant conditions. The increase in the leakage limit for the MSIVs from 6.0 scfh to 24.0 scfh will not inhibit the MSIVs' isolation function. Therefore, operation with the proposed increase in the MSIV leakage will not create the possibility of a new or different kind of accident from any accident previously evaluated.

The operation of Nine Mile Point Unit 2, in accordance with the proposed amendment, will not involve a significant reduction in a margin of safety.

The revised radiological analysis follows the very conservative fuel failure and instantaneous release assumptions of RG [Regulatory Guide] 1.3, with the exception of regulatory position C.1.f as permitted by SRP [Standard Review Plan] Section 6.5.5, ''Pressure Suppression Pool as a Fission Product Cleanup.'' The Staff approved the use of SRP Section 6.5.5. as part of the licensing basis of NMP2 in License Amendment No. 56.

The revised radiological analysis incorporates the maximum allowable leakage limit of 24.0 scfh for each of the four main steam lines. The revised radiological analysis also includes the impacts of the proposed power uprate of NMP2 and License Amendment No. 56. The new doses from the revised radiological analysis remain below the Staff acceptance criteria of 10 CFR [Part] 100 guideline values and GDC 19 (see Table 1 [of December 13, 1994, amendment request]). Therefore, operation with the proposed changes to the Technical Specifications will not significantly reduce a margin of safety.

Accordingly, as determined by the analysis above, this proposed amendment involves no significant hazards consideration.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

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NRC Project Director: Michael J. Case.