requirement acceptance criteria does not physically modify the plant nor does it modify the operation of any existing equipment.

3. The proposed change does not involve a significant reduction in the margin of safety. The change in vacuum acceptance criteria results in a slightly lower wind speed that may result in exfiltration from the building. However, this wind speed (31 mph) is in the realm of wind speeds which are infrequent at Plant Hatch. Furthermore, there are numerous conservatisms in the existing dose calculations including: neutral to stable meteorological conditions, ground level release until establishment of the required vacuum, accident source terms at event initiation, and no credit for plateout. The secondary containment would be maintained at a slight negative pressure shortly after the Standby Gas Treatment fans are running and the releases would be from the main stack (well before the accident source term would be present in the secondary containment). Some plateout would also occur and this is conservatively ignored. Therefore the margin of safety is not significantly reduced.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Appling County Public Library, 301 City Hall Drive, Baxley, Georgia 31513

Attorney for licensee: Ernest L. Blake, Jr., Esquire, Shaw, Pittman, Potts and Trowbridge, 2300 N Street, NW., Washington, DC 20037

NRC Project Director: Herbert N. Berkow

GPU Nuclear Corporation, et al., Docket No. 50-289, Three Mile Island Nuclear Station, Unit No. 1, Dauphin County, Pennsylvania

Date of amendment request: May 17, 1995

Description of amendment request: The proposed license amendment would revise Section 3.2 of the Technical Specifications (TSs) for Three Mile Island Nuclear Station, Unit 1 (TMI-1) to relocate the requirements for volume and boron concentration of the chemical addition system boric acid mix tank and the reclaimed boric acid storage tank from the TMI-1 TSs to the TMI-1 Core Operating Limits Report. The licensee, in its request, stated that the proposed changes are consistent with the intent of NRC Generic Letter 88-16.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Operation of the facility in accordance with the proposed amendment would not involve a significant increase in the probability of occurrence or consequences of an accident previously evaluated. The proposed amendment relocates chemical addition tank volume and boron concentration parameters from Technical Specifications to the TMI-1 Core Operating Limits Report. The proposed amendment provides continued control of the values of these parameters and assures these values are developed using NRC-approved reload methodologies consistent with all applicable limits of the safety analyses addressed in the TMI-1 [Final Safety Analysis Report] FSAR. The Technical Specifications retain the requirement to maintain the plant within the appropriate bounds of these limits. Therefore, the proposed amendment has no effect on the probability of occurrence or consequences of an accident previously evaluated.

2. Operation of the facility in accordance with the proposed amendment would not create the possibility of a new or different kind of accident from any accident previously evaluated. The proposed amendment relocates chemical addition tank volume and boron concentration parameters to the TMI-1 Core Operating Limits Report. The Technical Specifications retain the requirement to maintain the boric acid mix tank and reclaimed boric acid storage tank volume and boron concentration parameters within the appropriate limits. Therefore, the proposed amendment has no effect on the possibility of creating a new or different kind of accident from any accident previously evaluated.

3. Operation of the facility in accordance with the proposed amendment would not involve a significant reduction in a margin of safety. The proposed amendment provides continued control of the boric acid mix tank and reclaimed boric acid storage tank volume and boron concentration parameters and assures these values remain consistent with all applicable limits of the safety analyses addressed in the TMI-1 FSAR. Therefore, it is concluded that operation of the facility in accordance with the proposed amendment does not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Law/Government Publications Section, State Library of Pennsylvania, (REGIONAL DEPOSITORY) Walnut Street and Commonwealth Avenue, Box 1601, Harrisburg, PA 17105.

Attorney for licensee: Ernest L. Blake, Jr., Esquire, Shaw, Pittman, Potts & Trowbridge, 2300 N Street, NW., Washington, DC 20037

NRC Project Director: Phillip F. McKee

GPU Nuclear Corporation, et al., Docket No. 50-289, Three Mile Island Nuclear Station, Unit No. 1, Dauphin County, Pennsylvania

Date of amendment request: May 24, 1995

Description of amendment request: The proposed license amendment would revise Table 4.1-1 of the Technical Specifications (TSs) for Three Mile Island Nuclear Station, Unit 1 (TMI-1) to revise the test frequency requirement for the source range nuclear instrumentation from 7 days before reactor startup to 6 months before startup.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. Operation of the facility in accordance with the proposed TSCR would not involve a significant increase in the probability of occurrence or the consequences of an accident previously evaluated.

The proposed revision to the Technical Specifications does not involve any physical changes to the plant, and it does not impact the safety analysis with respect to design basis events and assumptions. The only change proposed is in the ''Test'' frequency for source-range Nuclear Instrumentation by revision of the appropriate Tech. Spec. tables. The revised testing requirement has no impact upon the probability of occurrence or the consequences of any accident previously evaluated, because no credit is taken in the accident analyses for the source range monitors nor are there any inputs to the Reactor Protection System. Tech. Spec. 3.1.9.2 requires that the control rod withdraw inhibit be operable at all times; however, it is not affected by this change request. Additionally, no nuclear safety equipment or systems interface with source-range nuclear instrumentation, and operator ability to monitor and trend post-accident neutron level is not affected by the proposed change. Therefore, this change request will not increase the probability of occurrence or the consequences of any previously analyzed accidents as described in the Updated [Final Safety Analysis Report] FSAR (UFSAR).

2. Operation of the facility in accordance with the proposed TSCR would not create the possibility of a new or different kind of accident from any accident previously evaluated.

The proposed revision to the TMI-1 Technical Specifications does not involve any physical changes to the plant, and does not impact on the safety analysis with respect to design basis events and assumptions. The only change proposed is in the ''Test'' frequency for Nuclear Instrumentation by revision of the appropriate Tech. Spec. tables. No nuclear safety equipment or