

review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room*

*location:* York County Library, 138 East Black Street, Rock Hill, South Carolina 29730

*Attorney for licensee:* Mr. Albert Carr, Duke Power Company, 422 South Church Street, Charlotte, North Carolina 28242

*NRC Project Director:* Herbert N. Berkow

**Duke Power Company, et al., Docket Nos. 50-413 and 50-414, Catawba Nuclear Station, Units 1 and 2, York County, South Carolina**

*Date of amendment request:* May 18, 1995, as supplemented by letter dated May 31, 1995.

*Description of amendment request:*

The proposed amendment would change Technical Specification (TS) 3.6.1.2 to defer the next scheduled containment integrated leak rate test (ILRT) at Catawba, Unit 2, for one outage, from the end-of-cycle (EOC) 7 refueling outage (scheduled for October 1995) to EOC-8 (scheduled for March 1997). Title 10 of the Code of Federal Regulations, Part 50, Appendix J, requires that three ILRTs be performed at approximately equal intervals during each 10-year service period at a nuclear station. "Approximately equal intervals" is defined in Catawba's TS as 40 plus or minus 10 months. The proposed one-time change would allow Catawba to extend that interval to less than or equal to 70 months.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The proposed change will not involve a significant increase in the probability or consequences of an accident previously evaluated.

Containment leak rate testing is not an initiator of any accident; the proposed interval extension does not affect reactor operations or accident analysis, and has no perceptible radiological consequences. Therefore, this proposed change will not involve a significant increase in the probability or consequences of any previously evaluated accident.

2. The proposed change will not create the possibility of any new accident not previously evaluated.

The proposed change does not affect normal plant operations or configuration, nor does it affect leak rate test methods. The test history at Catawba (no ILRT [intergrated leak

rate test] failures) provides continued assurance of the leak tightness of the containment structure.

3. There is no significant reduction in a margin of safety.

It has been documented in draft NUREG-1493 that an increase in the ILRT interval from 1 test every 3 years to 1 test every 10 years would result in an increase in population exposure risk in the vicinity of 5 representative plants from .02% to .14%. The proposed change included herein, an increase from 40 [plus or minus] 10 months to [less than or equal to] 70 months, represents a small fraction of that already very small increase in risk. Therefore, it may be concluded that no significant reduction in a margin of safety will occur.

Based on the above, no significant hazards consideration is created by the proposed change.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room*

*location:* York County Library, 138 East Black Street, Rock Hill, South Carolina 29730

*Attorney for licensee:* Mr. Albert Carr, Duke Power Company, 422 South Church Street, Charlotte, North Carolina 28242

*NRC Project Director:* Herbert N. Berkow

**Duke Power Company, Docket Nos. 50-369 and 50-370, McGuire Nuclear Station, Units 1 and 2, Mecklenburg County, North Carolina**

*Date of amendment request:* April 12, 1995

*Description of amendment request:*

The amendments delete Technical Specification 3/4.3.4, "Turbine Overspeed Protection," and its associated Bases. The deletion of TS 3/4.3.4 and its associated Bases provides Duke Power Company the flexibility to implement the manufacturer's recommendations for turbine steam valve surveillance test requirements. These test requirements will be relocated from the TS to the Selected Licensee Commitments (SLC) Manual. The SLC Manual is Chapter 16 of the Updated Final Safety Analysis Report.

*Basis for proposed no significant hazards consideration determination:* As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

*Criterion 1*

The requested amendments will not involve a significant increase in the

probability or consequences of an accident previously evaluated. Relocation of the affected TS section to the SLC Manual will have no effect on the probability of any accident occurring. In addition, the consequences of an accident will not be impacted since the above system will continue to be utilized in the same manner as before. No impact on the plant response to accidents will be created.

*Criterion 2*

The requested amendments will not create the possibility of a new or different kind of accident from any accident previously evaluated. No new accident causal mechanisms will be created as a result of relocating the affected TS requirements to the SLC Manual. Plant operation will not be affected by the proposed amendments and no new failure modes will be created.

*Criterion 3*

The requested amendments will not involve a significant reduction in a margin of safety. No impact upon any plant safety margins will be created. Relocation of the affected TS requirements to the SLC Manual is consistent with the content of the Westinghouse RSTS [Revised Standard Technical Specifications], as the NRC did not require technical specification controls for the turbine overspeed protection system in the RSTS. The proposed amendments are consistent with the NRC philosophy of encouraging utilities to propose amendments that are consistent with the content of the RSTS.

Based upon the preceding analyses, Duke Power Company concludes that the requested amendments do not involve a significant hazards consideration.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

*Local Public Document Room*

*location:* Atkins Library, University of North Carolina, Charlotte (UNCC Station), North Carolina 28223

*Attorney for licensee:* Mr. Albert Carr, Duke Power Company, 422 South Church Street, Charlotte, North Carolina 28242

*NRC Project Director:* Herbert N. Berkow

**Florida Power and Light Company, et al., Docket No. 50-335, St. Lucie Plant, Unit No. 1, St. Lucie County, Florida**

*Date of amendment request:* May 17, 1995

*Description of amendment request:*

The amendment will extend the applicability of the current Reactor Coolant System (RCS) Pressure/Temperature Limits and maximum allowed RCS heatup and cooldown rates to 23.6 Effective Full Power Years (EFPY) of operation. In addition, administrative changes are proposed for