The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Chattanooga-Hamilton County Library, 1101 Broad Street, Chattanooga, Tennessee 37402.

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 11H, Knoxville, Tennessee 37902.

NRC Project Director: Frederick J. Hebdon.

Wolf Creek Nuclear Operating Corporation, Docket No. 50–482, Wolf Creek Generating Station, Coffey County, Kansas

Date of amendment request: March 24, 1995.

Description of amendment request: This amendment request proposes to revise Technical Specification (TS) 1.7, "Containment Integrity," TS 3/4.6.1, "Containment Integrity," TS 3/4.6.3, "Containment Isolation Valves," and the associated Bases. These proposed changes will relocate TS Table 3.6–1, "Containment Isolation Valves," to Wolf Creek Generating Station procedures. This proposed change is in accordance with the guidance provided in Generic Letter 91–08, "Removal of Component Lists from Technical Specifications," dated May 6, 1991.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

(1) The proposed change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes simplify the technical specifications, meet the regulatory requirements for control of containment isolation, and are consistent with the guidelines of GL 91–08. The procedural details of Technical Specification Table 3.6–1 have not been changed, but only relocated to a different controlling document. The proposed changes are administrative in nature, should result in improved administrative practices, and do not affect plant operations.

The probability of occurrence of a previously evaluated accident is not increased because this change does not introduce any new potential accident initiating conditions. The consequences of an accident previously evaluated is not increased because the ability of [the] containment to restrict the release of any fission product radioactivity to the environment will not be degraded by this change.

(2) The proposed change does not create the possibility of a new or different kind of accident from any previously evaluated.

The proposed changes are administrative in nature, do not result in physical alterations or changes to the operation of the plant, and cause no change in the method by which any safety-related system performs its function. Therefore, this proposed change will not create the possibility of a new or different kind of accident from any previously evaluated.

(3) The proposed change does not involve a significant reduction in the margin of safety.

The administrative change to relocate Technical Specification Table 3.6–1 to appropriate plant procedures does not alter the basic regulatory requirements for containment isolation and will not adversely affect containment isolation capability for credible accident scenarios. Adequate control of the content of the table is assured by existing plant procedures.

The proposed relocation of Technical Specification Table 3.6–1 does not alter the requirements for containment isolation valve operability currently in the technical specifications. The LCO [limiting condition for operation] and Surveillance Requirements would be retained in the revised technical specifications. Therefore, the proposed change will not affect the meaning. application, and function of the current technical specification requirements for the valves in Table 3.6–1.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room locations: Emporia State University, William Allen White Library, 1200 Commercial Street, Emporia, Kansas 66801 and Washburn University School of Law Library, Topeka, Kansas 66621.

Attorney for licensee: Jay Silberg, Esq., Shaw, Pittman, Potts and Trowbridge, 2300 N Street, N.W., Washington, D.C. 20037.

NRC Project Director: William H. Bateman.

Previously Published Notices of Consideration of Issuance of Amendments to Facility Operating Licenses, Proposed No Significant Hazards Consideration Determination, and Opportunity for a Hearing

The following notices were previously published as separate individual notices. The notice content was the same as above. They were published as individual notices either because time did not allow the Commission to wait for this biweekly notice or because the action involved exigent circumstances. They are repeated here because the biweekly notice lists all amendments issued or proposed to be issued involving no significant hazards consideration.

For details, see the individual notice in the **Federal Register** on the day and page cited. This notice does not extend the notice period of the original notice.

Entergy Operations, Inc., Docket No. 50– 382, Waterford Steam Electric Station, Unit 3, St. Charles Parish, Louisiana

Date of amendment request: April 4, 1995, as supplemented by letter dated April 5, 1995.

Description of amendment request: The proposed amendment would change the technical specifications on moderator temperature coefficient. The proposed change constitutes a one time deviation not to perform the two-thirds end-of-cycle moderator temperature coefficient test for Cycle 7.

Date of individual notice in the Federal Register: April 11, 1995 (60 FR 18432).

Expiration date of individual notice: May 11, 1995.

Local Public Document Room location: University of New Orleans Library, Louisiana Collection, Lakefront, New Orleans, Louisiana 70122.

Notice of Issuance of Amendments to Facility Operating Licenses

During the period since publication of the last biweekly notice, the Commission has issued the following amendments. The Commission has determined for each of these amendments that the application complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations. The Commission has made appropriate findings as required by the Act and the Commission's rules and regulations in 10 CFR Chapter I, which are set forth in the license amendment.

Notice of Consideration of Issuance of Amendment to Facility Operating License, Proposed No Significant Hazards Consideration Determination, and Opportunity for A Hearing in connection with these actions was published in the **Federal Register** as indicated.

Unless otherwise indicated, the Commission has determined that these amendments satisfy the criteria for categorical exclusion in accordance with 10 CFR 51.22. Therefore, pursuant to 10 CFR 51.22(b), no environmental impact statement or environmental assessment need be prepared for these amendments. If the Commission has