1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change is temporary and allows a one-time extension of the ice condenser Surveillance Requirement 4.6.5.1.d for Cycle 7 to allow surveillance testing to coincide with the seventh refueling outage. The proposed surveillance interval extension will not cause a significant reduction in system reliability nor affect the ability of the system to perform the design function. Current monitoring of plant conditions and continuation of the surveillance testing required during normal plant operation will continue to be performed to ensure conformance with TS operability requirements. Therefore, this change does not involve a significant increase in the probability or consequences of an accident previously evaluated.

2. Create the possibility of a new or different kind of accident from any previously analyzed.

Extending the surveillance interval for the performance of ice condenser testing will not create the possibility of a new or different kind of accident. No change is required to any system configuration, plant equipment, or analyses.

3. Involve a significant reduction in a margin of safety.

The safety limits assumed in the accident analyses and the design function of the equipment required to mitigate the consequences of postulated accidents will not be changed significantly. Existing analysis indicates that the potential reduction in ice weight resulting from the proposed extension will continue to maintain the maximum containment accident pressure below 12 pounds per square inch gauge. The ice condenser will continue to support accident mitigation functions although some Row 1 baskets could drop slightly below the required 993-pound analysis limit. Therefore, the plant will be maintained with acceptable ice weights for accident mitigation and the proposed extension will not significantly reduce the margin of safety.

The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Chattanooga-Hamilton County Library, 1101 Broad Street, Chattanooga, Tennessee 37402.

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 11H, Knoxville, Tennessee 37902.

NRC Project Director: Frederick J. Hebdon.

Tennessee Valley Authority, Docket Nos. 50–327 and 50–328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of amendment request: April 6, 1995 (TS 95–11).

Description of amendment request: The proposed change would relocate the constant numerical value found in the overtemperature delta temperature and overpower delta temperature equations of Technical Specifications Table 2.2-1 and place them in the Core Operating Limits Report (COLR). This would be accomplished by revising notes 1 and 2 in Table 2.2-1 to state that the values are located in the COLR. The values of the constants, however, would not be changed. Also, the "Overtemperature and Overpower Delta Temperature Setpoint Parameter Values for Specification 2.2.1" would be added to the list of core operating limits specified in Section 6.9.1.14 that are required to be included in the COLR. In addition, a reference to WCAP-8745-P-A, "Design Bases for the Thermal Overpower delta-T and Thermal Overtemperature delta-T Trip Functions," would be added to Section 6.9.1.14.a.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

TVA has evaluated the proposed technical specification (TS) change and has determined that it does not represent a significant hazards consideration based on criteria established in 10 CFR 50.92(c). Operation of Sequoyah Nuclear Plant (SQN) in accordance with the proposed amendment will not:

1. Involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed changes will allow changes to the constant numerical values for the overtemperature delta temperature (OT[delta-]T) and overpower delta temperature (OP[delta-]T) equations in accordance with the 10 CFR 50.59 requirements. This revision does not revise these constants, but relocates them to the core operating limits report (COLR) that is governed by the 10 CFR 50.59 requirements. The addition of the lag compensator functions for measured [delta-]T and average temperature in these equations does not alter the setpoint because this lag function has a value of unity. Therefore, the proposed revision does not alter plant functions or setpoints, but does allow for a more timely revision process for parameters that may require changes due to specific fuel cycle requirements or updated plant analyses. The use of the lag functions and revisions to the constant numerical values will be maintained within the safety analysis for the plant by the 10 CFR 50.59 process requirements. The probability of an accident is not increased because the plant functions are not altered by the proposed revision and future changes will be in accordance with 10 CFR 50.59. Additionally, the consequences of an accident are not increased because the mitigation functions of the OT[delta-]T and

OP[delta-]T functions are not changed and revisions to the equations that derive the setpoints will be processed under the requirements of the 10 CFR 50.59 program.

2. Create the possibility of a new or different kind of accident from any previously analyzed.

The proposed revision will not change plant functions and future revisions will continue to be controlled in accordance with the 10 CFR 50.59 requirements. The addition of the lag functions does not create a new accident potential because these functions have already been considered in the analysis as shown in NUREG 1431. Therefore, the possibility of a new or different kind of accident is not created by the proposed revision.

3. Involve a significant reduction in a margin of safety.

Plant parameters are not altered by the proposed revision and the OT[delta-]T and OP[delta-]T functions will not reflect a change in setpoint generation or value. The proposed change will allow revision of the constant numerical values and use of the lag compensator functions in accordance with the 10 CFR 50.59 provisions to ensure the design basis of the plant is maintained. This revision does not result in changes that reduce the margin of safety because the OT[delta-]T and OP[delta-]T functions remain unchanged and future revisions to these functions will be performed in accordance with 10 CFR 50.59.

The NRC has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Chattanooga-Hamilton County Library, 1101 Broad Street, Chattanooga, Tennessee 37402.

Attorney for licensee: General Counsel, Tennessee Valley Authority, 400 West Summit Hill Drive, ET 11H, Knoxville, Tennessee 37902. NRC Project Director: Frederick J.

Hebdon.

Tennessee Valley Authority, Docket Nos. 50–327 and 50–328, Sequoyah Nuclear Plant, Units 1 and 2, Hamilton County, Tennessee

Date of amendment request: April 6, 1995 (TS 94–15).

Description of amendment request: The proposed change would modify the Technical Specifications associated with the Post Accident Sampling (PAS) system by deleting License Condition 2.C.(23)F for Unit 1 and 2.C.(16)g for Unit 2 that require operation of the PAS system in accordance with referenced letters no later than startup from the second refueling outage. The submittal also includes a revised description of operation of the PAS system for insertion into the Updated Final Safety