Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1)(i)–(v) and 2.714(d).

For further details with respect to this action, see the application for amendment which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room for the particular facility involved.

Carolina Power & Light Company, et al., Docket No. 50–400, Shearon Harris Nuclear Power Plant, Unit 1, Wake and Chatham Counties, North Carolina

Date of amendment request: March 20, 1995.

Description of amendment request: The licensee proposes a revision to Technical Specification (TS) 2.2.1, Reactor Trip System Instrumentation Setpoints, and to relocate cycle specific Overpower and Overtemperature Delta T trip setpoint parameters to the Core Operating Limits Report (COLR).

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

This change does not involve a significant hazards consideration for the following reasons:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

The proposed change of relocating Overtemperature Delta T * * * and the Overpower Delta T * * * trip setpoint parameters to the COLR has no influence or impact to the probability or consequences of an accident. The revised TS will continue to implement the Reactor Trip System Instrumentation [Overtemperature Delta T] and [Overpower Delta T] setpoint limits through reference to the parameters in the COLR. In addition, the COLR is subject to the existing controls of TS 6.9.1.6, including the establishment of the parameter values using an NRC approved methodology. Given that this change administratively relocates the selected trip setpoint parameter values to another TS-controlled document, there would be no increase in the probability or consequences of an accident previously evaluated.

2. The proposed amendment does not create the possibility of a new or different

kind of accident from any accident previously evaluated.

No safety-related equipment, safety function, or plant operation will be altered as a result of this proposed change. The limits are simply being relocated to another TScontrolled document. The TS will continue to require operation within the required limits as established per NRC approved methodologies. Therefore, the proposed changes do not create the possibility of a new or different kind of accident from any accident previously evaluated.

3. The proposed amendment does not involve a significant reduction in the margin of safety.

Relocation of the Reactor Trip System Instrumentation [Overtemperature Delta T] and [Overpower Delta T] setpoint limits to the TS-controlled COLR has no effect on the trip system setpoints currently in force in TS 2.2.1. Future revisions to the trip setpoint parameters are governed by TS 6.9.1.6. TS 6.9.1.6 lists each TS that references values in the COLR and the NRC approved methodologies utilized in developing those values. Since this change is only an administrative relocation of the selected trip setpoint parameter values to another TS controlled document, the proposed changes do not involve a significant reduction in a margin of safety.

The NRC staff has reviewed the licensee's analysis and, based on this review, it appears that the three standards of 10 CFR 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment request involves no significant hazards consideration.

Local Public Document Room location: Cameron Village Regional Library, 1930 Clark Avenue, Raleigh, North Carolina 27605.

Attorney for licensee: R. E. Jones, General Counsel, Carolina Power & Light Company, Post Office Box 1551, Raleigh, North Carolina 27602.

NRC Project Director: David B. Matthews.

Carolina Power & Light Company, et al., Docket No. 50–400, Shearon Harris Nuclear Power Plant, Unit 1, Wake and Chatham Counties, North Carolina

Date of amendment request: March 30, 1995.

Description of amendment request: The licensee proposes to revise the Emergency Diesel Generator (EDG) surveillance requirements contained in Technical Specification (TS) 4.8.1.1.2 to be consistent with NUREG–1431, Standard Technical Specifications for Westinghouse Plants, and to eliminate the need for duplicate EDG testing that has already been implemented to satisfy the requirements of the Station Blackout Rule and the Maintenance Rule.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

This change does not involve a significant hazards consideration for the following reasons:

1. The proposed amendment does not involve a significant increase in the probability or consequences of an accident previously evaluated.

A failure of the Emergency Diesel Generator (EDG) is not an initiator for any previously evaluated FSAR Chapter 15 accident scenario. By committing to and implementing an EDG reliability program that satisfies the requirements of the Station Blackout Rule and the Maintenance Rule, the Shearon Harris Nuclear Power Plant (SHNPP) will continue to ensure that target EDG reliability and availability is being achieved by conducting appropriate monitoring, testing, and maintenance activities. This program will be developed and controlled as a Plant Operating Manual procedure and will incorporate industry, vendor, and TDI Owners Group recommendations. Therefore, with commensurate levels of testing and inspection in place to provide assurance that the EDGs will perform their intended safety function in the event of an accident, the proposed changes will have no effect on the probability or consequences of such an accident.

2. The proposed amendment does not create the possibility of a new or different kind of accident from any accident previously evaluated.

A failure of the EDG is not an initiator for any previously evaluated FSAR Chapter 15 accident scenario nor would the proposed changes to the EDG surveillance requirements result in the possibility of a new or different kind of accident from any accident previously evaluated. By committing to and implementing an EDG reliability program that satisfies the requirements of the Station Blackout Rule and the Maintenance Rule, SHNPP will continue to ensure that target EDG reliability and availability is being achieved by conducting appropriate monitoring, testing, and maintenance activities. This program will be developed and controlled as a Plant Operating Manual procedure and will incorporate industry, vendor, and TransAmerica Delaval Inc. Owners Group recommendations. Therefore, with commensurate levels of testing and inspection in place to provide assurance that the EDGs will perform their intended safety function in the event of an accident, the proposed changes would not increase the possibility of a new or different kind of accident from any previously evaluated.

3. The proposed amendment does not involve a significant reduction in the margin of safety.

The proposed changes will not affect any parameters which relate to the margin of safety as defined in the Technical Specifications or the FSAR. Testing, inspection and maintenance necessary to verify the EDGs' ability to perform their intended safety function will continue to be