inputs from a variety of sources (*e.g.*, leakage from component cooling water system, reactor coolant system, condensate and feedwater system, turbine plant cooling water system, and auxiliary steam system). However, leakages from these systems are not expected to increase significantly since the operating pressures of these systems are either being maintained constant or are being increased only slightly due to the proposed power uprate.

The largest single source of liquid radioactive waste is from the ultrasonic cleaning of the condensate demineralizers. These demineralizers remove activated corrosion products which are expected to increase proportionally to the proposed power uprate. However, the total volume of processed waste is not expected to increase significantly, since the only appreciable increase in processed waste will be due to the slightly more frequent cleaning of these demineralizers. Based on a review of plant effluent reports and the slight increase expected due to the proposed power uprate, the NRC staff has concluded that the slight increase in the processing of liquid radioactive wastes will not have a significant increase in environment impact and that requirements of 10 CFR part 20 and 10 CFR part 50, appendix I, will continue to be met.

Gaseous radioactive effluents are produced during both normal operation and abnormal operation occurrences. These effluents are collected, controlled, processed, stored, and disposed of by the gaseous radioactive waste management systems which include the various building ventilation systems, the offgas system, and the standby gas treatment system (SGTS). The concentration of radioactive gaseous effluents released through the building ventilation systems during normal operation is not expected to increase significantly due to the proposed power uprate since the amount of fission products released into the reactor coolant (and subsequently into the building atmosphere) depends on the number and nature of fuel rod defects and is not dependent on reactor power level. The concentration of activation products contained in the reactor coolant is expected to remain unchanged, since the linear increase in the production of these activation products will be offset by the linear increase in steaming rate. Therefore, based on its review of the various building ventilation systems, the NRC staff has concluded that there will not be a significant adverse effect on airborne radioactive effluents as a result of the proposed power uprate.

Radiolysis of the reactor coolant causes the formation of hydrogen and oxygen, the quantities of which increase linearly with core power. These additional quantities of hydrogen and oxygen would increase the flow to the recombiners by 4.3 percent during uprated power conditions. The offgas system was originally designed for 105 percent of warranted steam flow which would not be exceeded during operation at the proposed uprated power level. Therefore, no changes will be required in the offgas system and since the offgas system will be operated within the originally evaluated design conditions, there will be no environmental impact that was not previously evaluated.

The SGTS is designed to minimize offsite radiation dose rates during venting and purging of both the primary and secondary containment atmosphere under accident or abnormal conditions. This is accomplished by maintaining the secondary containment at a slightly negative pressure (more negative than or equal to -0.25 inch water gauge) with respect to the outside atmosphere and discharging the secondary containment atmosphere through high-efficiency particulate air (HEPA) filters and charcoal absorbers. As noted in the Updated Safety Analysis Report (USAR), the SGTS charcoal absorbers are designed for a charcoal loading capacity of 10 mgI/gC and get the design requirements for 30-day and 100-day loss-of-coolant accident (LOCA) scenarios. The proposed power uprate would increase the post-LOCA iodine loading by 4.3 percent but the charcoal loading would still remain within the 10 mgI/gC loading and therefore, there would be no significant increase in environmental impact.

The licensee has evaluated the effects of the power uprate on in-plant radiation levels in the NMP-2 facility during both normal operation and postaccident. The licensee has concluded that radiation levels during both normal operation and post-accident may increase slightly (at most, proportional to the increase in power level). The slight increases in in-plant radiation levels expected due to the proposed power uprate are not expected to affect radiation zoning or shielding requirements. Individual worker occupational exposures will be maintained with acceptable limits by the existing as low as is reasonably achievable (ALARA) program which the licensee uses to control access to radiation areas. Therefore, the NRC staff has concluded that the slightly increased in-plant radiation levels will not have a significant environmental impact.

The offsite doses associated with normal operation are not significantly affected by operation at the proposed uprated power level and are expected to remain well within the limits of 10 CFR part 20 and 10 CFR part 50, appendix I. These limits are imposed by Technical Specifications 3/4.11.1, 3/4.11.2, 3/ 4.11.3, and 3/4.11.4, which will not be changed by the proposed power uprate. Therefore, the NRC staff has concluded that the offsite doses due to normal operation at the proposed power uprate conditions will not result in a significant environmental impact.

The dose evaluations for design basis accidents were performed for issuance of the current operating license based on 105 percent of the current rated power level. The proposed power uprate would be within the assumptions used during original licensing of the plant and, therefore, there will be no increase in environmental impacts over those evaluated in the NRC staff's Final Environmental Statement related to the operation of Nine Mile Point Nuclear Station, Unit No. 2 (NUREG–1085), May 1985.

The NRC staff has concluded that the NRC's FES (NUREG–1085) is valid for operation at the proposed uprated power conditions. The NRC staff also concluded that the plant operating parameters impacted by the proposed uprate would remain within the bounding conditions on which the conclusions of the FES are based.

The NRC staff has reviewed the licensee's reevaluation of the potential radiological and nonradiological environmental impacts for the proposed action. On the basis of this review, the NRC staff finds that the radiological and nonradiological environmental impacts associated with the proposed small increase in power are essentially immeasurable and do not change the conclusion in the FES that the operation of NMP-2 would cause no significant adverse impact upon the quality of the human environment.

Accordingly, the Commission concludes that this proposed action would result in no significant radiological or nonradiological environmental impact.

Alternative to the Proposed Action

Since the Commission has concluded there is no measurable environmental impact associated with the proposed action, any alternative with equal or greater impact need not be evaluated. The principal alternative would be to deny the requested amendment. Denial would not significantly reduce the environmental impact of plant operations, but would restrict operation