significant hazards consideration, the Commission may issue the amendment and make it immediately effective, notwithstanding the request for a hearing. Any hearing held would take place after issuance of the amendment.

If the final determination is that the amendment request involves a significant hazards consideration, any hearing held would take place before the issuance of any amendment.

A request for a hearing or a petition for leave to intervene must be filed with the Secretary of the Commission, U.S. Nuclear Regulatory Commission, Washington, DC 20555, Attention: Docketing and Services Branch, or may be delivered to the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington DC, by the above date. Where petitions are filed during the last 10 days of the notice period, it is requested that the petitioner promptly so inform the Commission by a toll-free telephone call to Western Union at 1-(800) 248-5100 (in Missouri 1-(800) 342-6700). The Western Union operator should be given Datagram Identification Number N1023 and the following message addressed to (Project Director): petitioner's name and telephone number, date petition was mailed, plant name, and publication date and page number of this Federal **Register** notice. A copy of the petition should also be sent to the Office of General Counsel, U.S. Nuclear Regulatory Commission, Washington, DC 20555, and to the attorney for the licensee.

Nontimely filings of petitions for leave to intervene, amended petitions, supplemental petitions and/or requests for a hearing will not be entertained absent a determination by the Commission, the presiding officer or the Atomic Safety and Licensing Board that the petition and/or request should be granted based upon a balancing of factors specified in 10 CFR 2.714(a)(1) (i)(v) and 2.714(d).

For further details with respect to this section, see the application for amendment which is available for public inspection at the Commission's Public Document Room, the Gelman Building, 2120 L Street, NW., Washington, DC, and at the local public document room for the particular facility involved.

Arizona Public Service Company, et al., Docket Nos. STN 50–528, STN 50–529, and STN 50–530, Palo Verde Nuclear Generating Station, Unit Nos. 1, 2, and 3, Maricopa County, Arizona

Date of amendment requests: December 7, 1994.

Description of amendment requests: The proposed amendment would revise the capacity of the ultimate heat sink (UHS) as described in the bases of Technical Specification 3/4.7.5, "Ultimate Heat Sink," from providing a 27-day cooling water supply to providing a 26-day cooling water supply. In addition, the reference to Regulatory Guide 1.27 in the bases of this TS would also be revised to reference the January 1976 revision rather than the March 1974 revision.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensees have provided their analysis about the issue of no significant hazards consideration, which is presented below:

Standard 1—Does the proposed change involve a significant increase in the probability or consequences of an accident previously evaluated?

The Essential spray pond system and the UHS do not initiate any accidents in Chapters 6 or 15 of the UFSAR [Updated Final Safety Analysis Report]. The justification and basis for the time that the UHS is available is not changed and continues to be consistent with the guidance in Regulatory Guide 1.27. The existing Technical Specification requirements and those components to which they apply are not altered by this Technical Specification amendment. Therefore, the change to the bases for Technical Specification 3/4.7.5 does not increase the probability of occurrence or the consequences of any previously evaluated accident.

Standard 2—Does the proposed change create the possibility of a new or different kind of accident from any accident previously evaluated?

The requirements for Technical Specification 3/4.7.5 are not changed. This amendment has no impact on plant maintenance, testing, shutdown equipment, or component qualification. Therefore, the possibility of a new or different kind of accident is not created by this amendment.

Standard 3—Does the proposed change involve a significant reduction in a margin of safety?

The change to the bases for Technical Specification 3/4.7.5 does not significantly alter existing Technical Specification requirements or those coponments to which they apply. The justification and basis for the time that the UHS is available without makeup is not changed and continues to be consistent with the guidance in Regulatory Guide 1.27. Regulatory Guide 1.27 states that "A capacity less than 30 days may be acceptable if it can be demonstrated that replenishment can be effected to ensure that continuous capability of the sink to perform its safety functions, taking into account the availability of replenishment equipment and limitations that may be imposed on "freedom of movement" following an accident." This change does not effect the continuous capability of the UHS to perform its safety function of providing decay heat removal

capability following an accident. The change updates the design basis of the UHS using more realistic conditions based on plant experience. Therefore, the change in the capacity of the UHS without makeup from 27 days to 26 days will not involve a significant reduction in margin of safety for the ultimate heat sink.

The NRC staff has reviewed the licensees' analysis and, based on that review, it appears that the three standards of 50.92(c) are satisfied. Therefore, the NRC staff proposes to determine that the amendment requests involve no significant hazards consideration.

Local Public Document Room location: Phoenix Public Library, 12 East McDowell Road, Phoenix, Arizona 85004.

Attorney for licensees: Nancy C. Loftin, Esq., Corporation Secretary and Counsel, Arizona Public Service Company, P.O. Box 53999, Mail Station 9068, Phoenix, Arizona 85072–3999.

NRC Project Director: Theodore R. Quay.

Carolina Power & Light Company, Docket No. 50–261, H. B. Robinson Steam Electric Plant, Unit No. 2, Darlington County, South Carolina

Date of amendment request: June 18, 1992, as supplemented December 8, 1992, and revised February 3, 1995.

Description of amendment request: The proposed Technical Specification (TS) amendment adds limiting conditions of operation and surveillance requirements for the pressurizer poweroperated relief valves (PORVs) and their associated block valves whenever average temperature (Tavg) is above 350 degrees F or the reactor is critical. Specifications have also been added for low-temperature overpressure protection whenever Tavg is less than 350 degrees F and the reactor coolant system is not vented to the containment. The February 3, 1995, revision made editorial changes to previous TS pages and made changes to conform with an additional provision of the guidance for surveillance testing of the block valves associated with the pressurizer PORVs. In addition, the licensee has requested an editorial change to TS page 3.1.-11 to revise the references to two figures that have been superseded.

Basis for proposed no significant hazards consideration determination: As required by 10 CFR 50.91(a), the licensee has provided its analysis of the issue of no significant hazards consideration, which is presented below:

1. The requested revision does not involve a significant increase in the probability or consequences of an accident previously