underdose and the patient returned on August 2, 1994, for the completion of the therapy. The patient was given a written report of the misadministration on August 9, 1994. The licensee submitted a report for the misadministration on August 10, 1994. NRC Region I has enlisted the services of a medical consultant to evaluate the clinical consequences of this misadministration and awaits his report.

A copy of NUREG-0090, Vol. 17, No. 3 is available for inspection or copying for a fee at the NRC Public Document Room, 2120 L Street NW. (Lower Level), Washington, D.C. 20037, or at any of the nuclear power plant Local Public Document Rooms throughout the country.

Copies of this report (or any of the previous reports in this series), may be purchased from the Superintendent of Documents, U.S. Government Printing Office, Post Office Box 37082, Washington, DC 20013–7082. A year's subscription to the NUREG–0090 series publication, which consists of four issues, is also available.

Copies of the report may also be purchased from the National Technical Information Service, U.S. Department of Commerce, 5285 Port Royal Road, Springfield, VA 22161.

Dated at Rockville, MD this 16th day of February, 1995.

For the Nuclear Regulatory Commission. **John C. Hoyle**,

Acting Secretary of the Commission. [FR Doc. 95–4382 Filed 2–22–95; 8:45 am] BILLING CODE 7590–01–M

## [Docket Nos. 50-352 and 50-353]

## In the Matter of: Philadelphia Electric Company (Limerick Generating Station, Units 1 and 2); Exemption

I.

Philadelphia Electric Company (the licensee), is the holder of Facility Operating License Nos. NPF–39 and NPF–85, which authorize operation of the Limerick Generating Station (LGS), Units 1 and 2. The licenses provide, among other things, that the licensee is subject to all rules, regulations, and orders of the Nuclear Regulatory Commission (the Commission) now and hereafter in effect. The facilities consist of two boiling water reactors located in Montgomery County, Pennsylvania.

## Ħ

Section 50.54(o) of 10 CFR Part 50 requires that primary reactor containments for water cooled power reactors be subject to the requirements of Appendix J to 10 CFR Part 50.

Appendix J contains the leakage test requirements, schedules, and acceptance criteria for tests of the leak tight integrity of the primary reactor containment and systems and components which penetrate the containment. Sections II.H.4 and III.C.2 of Appendix J to 10 CFR Part 50 require leak rate testing of Main Steam Isolation Valves (MSIVs) at the calculated peak containment pressure related to the design basis accident, and Section III.C.3 requires that the measured leak rates be included in the combined local leak rate test results. The proposed deletion of the MSIV Leakage Control System (LCS), and proposed use of an alternate leakage pathway affects the description of an existing exemption which allows the leak rate testing of the MSIVs at a reduced pressure and the exclusion of the measured leakage from the combined local leak rate test results. The original exemption is contained in the LGS Safety Evaluation Report (SER) (NUREG-0991, and its Supplement 3).

By letter dated December 22, 1994, the licensee requested an exemption from the Commission's regulations. The subject exemption is from the requirements of 10 CFR Part 50, Appendix J, "Primary Reactor Containment Leakage Testing for Water-Cooled Power Reactors," Sections II.H.4, III.C.2, and III.C.3, to allow alternative testing pressure and leakage limits for the MSIVs and to exclude MSIV leakage from the combined local leak rate test results after deletion of the LCS.

The staff issued for LGS, Units 1 and 2, the current exemption from 10 CFR Part 50, Appendix J, Sections II.H.4, III.C.2, and III.C.3, based on the conclusion that the LGS, Units 1 and 2, MSIV leak testing methods were acceptable alternatives to the requirements. This conclusion was included in the LGS SER (NUREG-0991, and its Supplement 3). The SER also described that in the event of a loss-ofcoolant-accident (LOCA), the MSIV LCS will maintain a negative pressure between the MSIV and the effluent will be discharged into a volume where it will be processed by the standby gas treatment system before being released to the environment. The licensee had performed a radiological analysis based on an assumed leak rate limit of 11.5 standard cubic feet per hour (scfh), and the MSIVs were planned to be periodically tested to ensure the validity of the radiological analysis. The staff concluded that the current LGS testing procedure, where two valves on one steam line are tested simultaneously, between the valves, utilizing a reduced test pressure (i.e., half a peak containment pressure of 22 psig applied

between the MSIVs) was acceptable. Also, the staff excluded the MSIV test leakage rate from the combined local leak rate because the MSIV leakage had been accounted for separately in the radiological analysis of the site.

By letter dated January 14, 1994, the licensee submitted a Technical Specifications (TS) amendment request for LGS, Units 1 and 2, which supports the planned modification to eliminate the MSIV LCS and utilize an alternate leakage pathway (main steam lines and condenser). This proposal is based on the Boiling Water Reactor Owners Group (BWROG) method summarized in General Electric Report NEDC-31858P. Revision 2, "BWROG Report for increasing MSIV Leakage Rate Limits and Elimination of Leakage Control System." Therefore, the description of the MSIV LCS and the assumed MSIV leak rate are no longer accurate once the proposed TS modification is performed and implemented.

The licensee's January 14, 1994, TS (amendment) request states that a plantspecific radiological analysis has been performed in accordance with NEDC-31858P, Revision 2, to assess the effects of the proposed increase to the allowable MSIV leakage rate in terms of Main Control Room (MCR) and off-site doses following a postulated design basis LOCA. This analysis utilizes the hold-up volume of the main steam piping and condenser as an alternate method for treating MSIV leakage. The radiological analysis uses standard conservative assumptions for the radiological source term consistent with Regulatory Guide (RG) 1.3, "Assumptions Used for Evaluating the Potential Radiological Consequences of a Loss-of-Coolant-Accident for Boiling Water Reactors," Revision 2, dated June 1974. The analysis results demonstrate that dose contributions from the proposed MSIV leakage rate limit of 100 scfh per MSIV, not to exceed 200 scfh for all four main steam lines, and considering the proposed deletion of the MSIV LCS, result in an acceptable increase to the LOCA doses previously evaluated against the regulatory limits for the off-site doses and MCR doses contained in 10 CFR Part 100, and 10 CFR Part 40, Appendix A, General Design Criteria (GDC) 19, respectively. The proposed calculated off-site and MCR doses resulting from a LOCA are the sum of the LOCA doses previously evaluated (currently described in the Updated Final Safety Analysis Report), and the additional doses calculated using the alternate MSIV leakage treatment method. The method of calculating the revised doses is conservative, since the LOCA doses